# PALOMARES NUCLEAR WEAPONS ACCIDENT 



REVISED DOSE EVALUATION REPORT

## Volume III <br> Appendix C. 2 - Repeat Analysis Cases Appendix C. 3 - Contamination Cutoff Cases Appendix C. 4 - Remaining Cases

Date: April 2001
Contract: GS-35F-4813G
Task Order: WFZ578410
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Prepared For: Radiation Protection Division
Air Force Medical Operations Agency Bolling AFB, DC 20332-7050

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Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 140,000 | $43 / 0.43$ |
| LUDEP | 383,000 | $27 / 0.27$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> $($ rem/Sv |
| :--- | :---: | :---: | :---: |
| Testes | $6.2 \mathrm{E}+00 / 6.2 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.6 \mathrm{E}+00 / 1.6 \mathrm{E}-02$ |
| Breast | $2.1 \mathrm{E}-04 / 2.1 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $3.1 \mathrm{E}-05 / 3.1 \mathrm{E}-07$ |
| Red Marrow | $3.4 \mathrm{E}+01 / 3.4 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $4.0 \mathrm{E}+00 / 4.0 \mathrm{E}-02$ |
| Lung | $1.6 \mathrm{E}+02 / 1.6 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.9 \mathrm{E}+01 / 1.9 \mathrm{E}-01$ |
| Thyroid | $2.0 \mathrm{E}-04 / 2.0 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $5.9 \mathrm{E}-06 / 5.9 \mathrm{E}-08$ |
| Bone Surface | $4.4 \mathrm{E}+02 / 4.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.3 \mathrm{E}+01 / 1.3 \mathrm{E}-01$ |
| Liver | $7.8 \mathrm{E}+017.8 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $4.7 \mathrm{E}+00 / 4.7 \mathrm{E}-02$ |
| Other | $7.4 \mathrm{E}+00 / 7.4 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $4.4 \mathrm{E}-01 / 4.4 \mathrm{E}-03$ |
| Lower Large Intestine | $1.6 \mathrm{E}-02 / 1.6 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $9.5 \mathrm{E}-04 / 9.5 \mathrm{E}-06$ |
| Upper Large Intestine | $5.3 \mathrm{E}-03 / 5.3 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $3.2 \mathrm{E}-04 / 3.2 \mathrm{E}-06$ |
| Small Intestine | $1.1 \mathrm{E}-03 / 1.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $6.5 \mathrm{E}-05 / 6.5 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $4.3 \mathrm{E}+01 / 4.3 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. The sample analyzed by gross alpha counting only was not included in the analysis since it was an on-site sample with a result $>0.1 \mathrm{pCi}$, leading to a suspicion of sample contamination. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $140,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $43 \mathrm{rem} / 0.43 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $383,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $27 \mathrm{rem}(0.27 \mathrm{~Sv})$.

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $140,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 43 rem ( 0.43 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level $(0.100 \mathrm{rem})$ for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$

Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

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## Internal Dosimetry Evaluation Form



| RECOMMIENDATIONS: |  |  |  |
| :---: | :---: | :---: | :---: |
| Additional Bioassay Required Suggested Sampling Frequency: | $\square$ Urinalysis | $\square$ Fecal | $\square$ In Vivo |
| Work Restrictions: N/A |  |  |  |

## Internal Dosimetry Case Narrative

## Identification:



## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $1 / 29 / 66$. The date is the midpoint of the period on station from $1 / 20 / 66$ to $2 / 7 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-442$ | G | $2 / 7 / 66$ | 1.24 | 0.950 |  |
| $66-3267$ | AS | $6 / 1 / 66$ | 0.091 | 0.065 | $\checkmark$ |
| $66-3267$ | $G$ | $6 / 1 / 66$ | NR | NR |  |

* G means gross alpha counting; AS means alpha spectrometry; NR means no result reported.

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

Modeling:
CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CINDY | 130,000 | $40 / 0.4$ |
| LUDEP | 370,000 | $26 / 0.26$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> (rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $5.8 \mathrm{E}+00 / 5.8 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.5 \mathrm{E}+00 / 1.5 \mathrm{E}-02$ |
| Breast | $1.9 \mathrm{E}-04 / 1.9 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $2.9 \mathrm{E}-05 / 2.9 \mathrm{E}-07$ |
| Red Marrow | $3.1 \mathrm{E}+01 / 3.1 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $3.8 \mathrm{E}+00 / 3.8 \mathrm{E}-02$ |
| Lung | $1.5 \mathrm{E}+02 / 1.5 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.8 \mathrm{E}+01 / 1.8 \mathrm{E}-01$ |
| Thyroid | $1.8 \mathrm{E}-04 / 1.8 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $5.4 \mathrm{E}-06 / 5.4 \mathrm{E}-08$ |
| Bone Surface | $4.1 \mathrm{E}+02 / 4.1 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.2 \mathrm{E}+01 / 1.2 \mathrm{E}-01$ |
| Liver | $7.2 \mathrm{E}+01 / 7.2 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $4.3 \mathrm{E}+00 / 4.3 \mathrm{E}-02$ |
| Other | $6.9 \mathrm{E}+00 / 6.9 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $4.1 \mathrm{E}-01 / 4.1 \mathrm{E}-03$ |
| Lower Large Intestine | $1.5 \mathrm{E}-021.5 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $8.9 \mathrm{E}-04 / 8.9 \mathrm{E}-06$ |
| Upper Large Intestine | $5.0 \mathrm{E}-03 / 5.0 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $3.0 \mathrm{E}-04 / 3.0 \mathrm{E}-06$ |
| Small Intestine | $1.0 \mathrm{E}-03 / 1.0 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $6.1 \mathrm{E}-05 / 6.1 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $4.0 \mathrm{E}+01 / 4.0 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. The sample analyzed by gross alpha counting only was not included in the analysis since it was an on-site sample with a result $>0.1 \mathrm{pCi}$, leading to a suspicion of sample contamination. The result was fit using CINDY and the Jones excretion model, to estimate an intake $(130,000 \mathrm{pCi})$, organ doses, and a CEDE ( $40 \mathrm{rem} / 0.4 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $370,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $26 \mathrm{rem}(0.26 \mathrm{~Sv}$ ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $130,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 40 rem ( 0.4 Sv). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$

Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$


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## FEB 101966



Internal Dosimetry Evaluation Form

| NAME：（1）（6） |  |  | SSN: |  |
| :---: | :---: | :---: | :---: | :---: |
| MODE OF INTAKE：  <br> 区 Inhalation  <br> $\square$ Ingestion  <br> $\square$ Unknown | $\square$ Injection $\square$ Absorption $\square$ Not applicable | INTAKE DATE OR PERIOD： 2／4／66 through $3 / 1 / 66$ ，onsite 2／16／66 |  |  |
| SUMMARY OF EXIPOSURE CONDITIONS： <br> Radionuclides／Respiratory Class／Particle Size：${ }^{239} \mathrm{Pu} / 100 \%$ Class $\mathrm{Y} / \mathrm{l} \mu \mathrm{m}$ AMAD <br> Date or Period of Evaluated Data： 1 sample，3／1／66 <br> Duration of Exposure：Unknown <br> Location of Exposure：Camp Wilson，near Palomares，Spain |  |  |  |  |
| EVALUATION DATA： <br> Air Sampling Health Physics Survey Data Bioassay－Urinalysis Fecal Nasal Smears In Vivo | $\square$ Attached $\square$ Attached $\boxed{\text { Attached }}$ $\square$ Attached $\square$ Attached $\square$ Attached | $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process | 《 Unavailable <br> 区 Unavailable <br> $\square$ Unavailable <br> 区 Unavailable <br> 《 Unavailable <br> 区 Unavailable |  |
| Medical Treatment： <br> Skin Decontamination： <br> Decorporation： <br> Catharsis： <br> Surgical excision： | $\square$ Yes $\square$ Yes $\square$ Yes $\square$ Yes | Q No Date： <br> Q No Agent <br> Х No Agent： <br> 区 No Date： |  | Date： <br> Date： |

## EVALUATION METHODOLOGY：

Assumptions：Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 16 / 66$

| Code／Model used for： | Intake Estimate：CINDY，Ver．1．4／JONES |
| :--- | :--- |
|  | Dose Estimate：$C$ CINDY，Ver．1．4／ICRP 30，Part 4，General Systemic Model |

RESULTS SUMMARY
Estimated Intake Activity（pCi）： 270000
50 YR CEDE（rem）： 83 （ 0.83 Sv ）

| Organ Dose Equivalent Summary | $\mathbf{5 0}$ YR CDE（rem／Sv） |
| :---: | :---: |
| Bone Surface | $840 / 8.4$ |
| Lung | $310 / 3.1$ |
| Liver | $150 / 1.5$ |
| Red Marrow | $65 / 0.65$ |
| Other | $14 / 0.14$ |
| Testes | $12 / 0.12$ |


| DOSE ASSESSOR： | DATE：＿＿＿ |  | PEER REVIEWER： |
| :--- | :--- | :--- | :--- |
| Signature： |  | Signature： |  |
| Print Name： | Print Name： |  |  |
| SSN： |  |  |  |



## Internal Dosimetry Case Narrative

## Identification:

## Name:

SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

Other Information:
None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 16 / 66$. The date is the midpoint of the period on station from 2/4/66 to 3/1/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1855$ | AS | $3 / 1 / 66$ | 0.294 | 0.007 | $\checkmark$ |
| $66-1855$ | G | 3/1/66 | 1.52 | 0.310 |  |
| G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

Modeling:
CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 270,000 | $83 / 0.83$ |
| LUDEP | $1,270,000$ | $89 / 0.89$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> $($ rem $/$ Sv $)$ |
| :--- | :---: | :---: | :---: |
| Testes | $1.2 \mathrm{E}+01 / 1.2 \mathrm{E}-01$ | $2.5 \mathrm{E}-01$ | $3.0 \mathrm{E}+00 / 3.0 \mathrm{E}-02$ |
| Breast | $4.0 \mathrm{E}-04 / 4.0 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $6.0 \mathrm{E}-05 / 6.0 \mathrm{E}-07$ |
| Red Marrow | $6.5 \mathrm{E}+01 / 6.5 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $7.8 \mathrm{E}+00 / 7.8 \mathrm{E}-02$ |
| Lung | $3.1 \mathrm{E}+02 / 3.1 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $3.7 \mathrm{E}+01 / 3.7 \mathrm{E}-01$ |
| Thyroid | $3.8 \mathrm{E}-04 / 3.8 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $1.1 \mathrm{E}-05 / 1.1 \mathrm{E}-07$ |
| Bone Surface | $8.4 \mathrm{E}+02 / 8.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $2.5 \mathrm{E}+01 / 2.5 \mathrm{E}-01$ |
| Liver | $1.5 \mathrm{E}+02 / 1.5 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $9.0 \mathrm{E}+00 / 9.0 \mathrm{E}-02$ |
| Other | $1.4 \mathrm{E}+01 / 1.4 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $8.6 \mathrm{E}-01 / 8.6 \mathrm{E}-03$ |
| Lower Large Intestine | $3.1 \mathrm{E}-02 / 3.1 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $1.8 \mathrm{E}-03 / 1.8 \mathrm{E}-05$ |
| Upper Large Intestine | $1.0 \mathrm{E}-02 / 1.0 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $6.2 \mathrm{E}-04 / 6.2 \mathrm{E}-06$ |
| Small Intestine | $2.1 \mathrm{E}-03 / 2.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $1.3 \mathrm{E}-04 / 1.3 \mathrm{E}-06$ |
| Effective Dose Equivalent |  |  | $8.3 \mathrm{E}+01 / 8.3 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting and alpha spectrometry. The gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $270,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $83 \mathrm{rem} / 0.83 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $1,270,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $89 \mathrm{rem}(0.89 \mathrm{~Sv})$.

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $270,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $83 \mathrm{rem}(0.83$ Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is more than the working lifetime limit of 50 rem
recommended by the National Council on Radiation Protection and Measurements (NCRP). These dose levels are significant, although they were based on very limited data. However, follow-up urine sampling should be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$


## Internal Dosimetry Evaluation Form

| NAME ${ }^{(10)(6)}$ | SSN：${ }^{\text {（b）（6）}}$ |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: |
| MODE OF INTAKE： <br> Inhalation Ingestion Unknown | Injection Absorption Not applicable | INTAKE DATE OR PERIOD： 1／18／66 through 2／7／66，onsite 1／28／66 |  |  |  |
| SUMMARY OF EXPOSURE CONDITIONS： <br> Radionuclides／Respiratory Class／Particle Size：${ }^{239} \mathrm{Pu} / 100 \%$ Class Y／1 $\mu \mathrm{m}$ AMAD <br> Date or Period of Evaluated Data： 2 samples，2／7／66 and 6／1／66 <br> Duration of Exposure：Unknown <br> Location of Exposure：Camp Wilson，near Palomares，Spain |  |  |  |  |  |
| EVALUATION DATA： <br> Air Sampling Health Physics Survey Data Bioassay－Urinalysis Fecal Nasal Smears In Vivo | $\square$ Attached $\square$ Attached $\boxed{\text { Attached }}$ $\square$ Attached $\square$ Attached $\square$ Attached | $\square \ln P$ $\square \ln P$ $\square \operatorname{In} P$ $\square \operatorname{In} P$ $\square \operatorname{In} P$ $\square \ln P$ | cess <br> acess <br> cess <br> cess <br> acess <br> cess | $\begin{aligned} & \text { Unavailable } \\ & \text { Q Unavailable } \\ & \square \text { Unavailable } \\ & \text { Q Unavailable } \\ & \boxtimes \text { Unavailable } \\ & \text { Q Unavailable } \end{aligned}$ |  |
| Medical Treatment： <br> Skin Decontamination： <br> Decorporation： <br> Catharsis： <br> Surgical excision： | $\square$ Yes $\square$ Yes $\square$ Yes $\square$ Yes | 《 No Х No Х No 区 No | Date： <br> Agent： <br> Agent： <br> Date： |  | Date： <br> Date： |

EVALUATION METHODOLOGY：
Assumptions：Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on $1 / 28 / 66$
Code／Model used for：Intake Estimate：CINDY，Ver．1．4／JONES
Dose Estimate：CINDY，Ver．1．4／ICRP 30，Part 4，General Systemic Model

## RESULTS SUMMARY

Estimated Intake Activity（pCi）： 68000
50 YR CEDE（rem）： 21 （ 0.21 Sv ）

| Organ Dose Equivalent Summary <br> Bone Surface <br> Lung <br> Liver <br> Red Marrow <br> Other <br> Testes | $\begin{gathered} 50 \text { YR CDE (rem/Sv) } \\ 210 / 2.1 \\ 78 / 0.78 \\ 38 / 0.38 \\ 16 / 0.16 \\ 3.6 / 0.036 \\ 3 / 0.03 \end{gathered}$ |  |
| :---: | :---: | :---: |
| DOSE ASSESSOR：DATE： | PEER REVIEWER： | DATE： |
| Signature： | Signature： |  |
| Print Name： | Print Name： |  |
| SSN： | SSN： |  |


| RECOMMIENDATIONS： |  |  |  |
| :--- | :--- | :--- | :--- | :--- |
| Additional Bioassay Required <br> Suggested Sampling Frequency： <br> Work Restrictions： <br> N／A | $\square$ Urinalysis | $\square$ Fecal | $\square$ In Vivo |

Suggested Sampling Frequency：
Work Restrictions：N／A

Internal Dosimetry Case Narrative

## Identification:

## Name: <br> SSN:



## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $\mathrm{l} / 28 / 66$. The date is the midpoint of the period on station from $1 / 18 / 66$ to $2 / 7 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-499$ | G | $2 / 71 / 66$ | 1.66 | 0.700 | $\checkmark$ |
| $66-3271$ | AS | $6 / 1 / 66$ | 0.047 | 0.047 | $\checkmark$ |
| $66-3271$ | G | $6 / 1 / 66$ | NR | NR |  |

* G means gross alpha counting; AS means alpha spectrometry; NR means no result reported.

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake $(\mathrm{pCi})$ | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 68,000 | $21 / 0.21$ |
| LUDEP | 191,000 | $13 / 0.13$ |

Doses to individual organs and estimation of the effective dose equivalent using CNDY reported the following results:

| Organ | Dose Equivalent <br> (rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $3.0 \mathrm{E}+00 / 3.0 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $7.6 \mathrm{E}-01 / 7.6 \mathrm{E}-03$ |
| Breast | $1.0 \mathrm{E}-04 / 1.0 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $1.5 \mathrm{E}-05 / 1.5 \mathrm{E}-07$ |
| Red Marrow | $1.6 \mathrm{E}+01 / 1.6 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $2.0 \mathrm{E}+00 / 2.0 \mathrm{E}-02$ |
| Lung | $7.8 \mathrm{E}+01 / 7.8 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $9.3 \mathrm{E}+00 / 9.3 \mathrm{E}-02$ |
| Thyroid | $9.5 \mathrm{E}-05 / 9.5 \mathrm{E}-07$ | $3.0 \mathrm{E}-02$ | $2.8 \mathrm{E}-06 / 2.8 \mathrm{E}-08$ |
| Bone Surface | $2.1 \mathrm{E}+02 / 2.1 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $6.4 \mathrm{E}+00 / 6.4 \mathrm{E}-02$ |
| Liver | $3.8 \mathrm{E}+01 / 3.8 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $2.3 \mathrm{E}+00 / 2.3 \mathrm{E}-02$ |
| Other | $3.6 \mathrm{E}+00 / 3.6 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $2.2 \mathrm{E}-01 / 2.2 \mathrm{E}-03$ |
| Lower Large Intestine | $7.7 \mathrm{E}-03 / 7.7 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $4.6 \mathrm{E}-04 / 4.6 \mathrm{E}-06$ |
| Upper Large Intestine | $2.6 \mathrm{E}-03 / 2.6 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $1.6 \mathrm{E}-04 / 1.6 \mathrm{E}-06$ |
| Small Intestine | $5.3 \mathrm{E}-04 / 5.3 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $3.2 \mathrm{E}-05 / 3.2 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $2.1 \mathrm{E}+01 / 2.1 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The sample analyzed by gross alpha counting only was not included in the analysis since it was an on-site sample with a result $>0.1 \mathrm{pCi}$, leading to a suspicion of sample contamination. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $68,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $21 \mathrm{rem} / 0.21 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $191,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $13 \mathrm{rem}(0.13 \mathrm{~Sv}$ ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $68,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 21 rem ( 0.21 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is less than half the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$




Internal Dosimetry Evaluation Form

\begin{tabular}{|c|c|c|}
\hline \multicolumn{2}{|l|}{NAME:(10) (6)} \& SSN: (1) (6) \\
\hline \begin{tabular}{cl}
\hline MODE OF INTAKE: \& \\
\& \begin{tabular}{l} 
Inhalation \\
\(\square\)
\end{tabular} \\
\& \(\square\) Injection \\
\& \(\square\) Unknown \\
Absorption \\
\& \(\square\) Not applicable
\end{tabular} \& INTAKE DA 1/20/66 throug 1/29/66 \& TE OR PERIOD: 2/7/66, onsite \\
\hline \multicolumn{3}{|l|}{\begin{tabular}{l}
SUMMARY OF EXPOSURE CONDITIONS: \\
Radionuclides/Respiratory Class/Particle Size: \({ }^{239} \mathrm{P} u / 100 \%\) Class Y/1 \(\mu \mathrm{m}\) AMAD \\
Date or Period of Evaluated Data: 2 samples, 2/7/66 and 6/1/66 \\
Duration of Exposure: Unknown \\
Location of Exposure: Camp Wilson, near Palomares, Spain
\end{tabular}} \\
\hline \multicolumn{3}{|l|}{\begin{tabular}{llll} 
EVALUATION DATA: \& \& \\
\begin{tabular}{llll} 
Air Sampling
\end{tabular} \& \(\square\) Attached \& \(\square\) In Process \& Q Unavailable \\
Health Physics Survey Data \& \(\square\) Attached \& \(\square\) In Process \& Q Unavailable \\
Bioassay - Urinalysis \& \(\boxed{4 t t a c h e d ~}\) \& \(\square\) In Process \& \(\square\) Unavailable \\
Fecal \& \(\square\) Attached \& \(\square\) In Process \& Q Unavailable \\
Nasal Smears \& \(\square\) Attached \& \(\square\) In Process \& Q Unavailable \\
In Vivo \& \(\square\) Attached \& \(\square\) In Process \& Q Unavailable
\end{tabular}} \\
\hline  \& \begin{tabular}{ll} 
Х No \& Date: \\
Х No \& Agent: \\
Х No \& Agent: \\
区 No \& Date:
\end{tabular} \& \begin{tabular}{l}
\(\qquad\) \\
Date:
\end{tabular} \\
\hline EVALUATION METHODOLOGY:
Assumptions: Acute inhalation intake of \({ }^{239} \mathrm{Pu}, 100 \%\)
Code/Model used for:

Intake Estimate: CINDY, V

Dose Estimate: CINDY, V \& \begin{tabular}{l}
lass $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMA <br>
er. 1.4/JONES <br>
er. 1.4/ICRP 30, Pa

 \& 

D particle size on 1/29/66 <br>
t 4, General Systemic Model
\end{tabular} <br>

\hline \multicolumn{3}{|l|}{| RESULTS SUMMARY |  |
| :--- | :---: |
| Estimated Intake Activity (pCi): 210000 |  |
| 50 YR CEDE (rem) : 65 ( 0.65 Sv) |  |
| Organ Dose Equivalent Summary | $\mathbf{5 0 ~ Y R ~ C D E ~ ( r e m / S v ) ~}$ |
| Bone Surface | $650 / 6.5$ |
| Lung | $240 / 2.4$ |
| Liver | $120 / 1.2$ |
| Red Marrow | $51 / 0.51$ |
| Other | $11 / 0.11$ |
| Testes | $9.4 / 0.094$ |} <br>


\hline | DOSE ASSESSOR: |
| :--- |
| DATE: $\qquad$ |
| Signature: $\qquad$ |
| Print Name: $\qquad$ |
| SSN: | \& | PEER REVII |
| :--- |
| Signature: |
| Print Nam |
| SSN: $\qquad$ | \& | WER: |
| :--- |
| DAT | <br>

\hline
\end{tabular}

| RECOMMENDATIONS: |  |  |  |  |
| :--- | :--- | :--- | :--- | :--- |
| Additional Bioassay Required <br> Suggested Sampling Frequency: <br> Work Restrictions:$\quad$ N/A |  | $\square$ Urinalysis | $\square$ Fecal | $\square$ In Vivo |
|  |  |  |  |  |

## Internal Dosimetry Case Narrative

## Identification:

Name:
SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; 100\% Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $1 / 29 / 66$. The date is the midpoint of the period on station from $1 / 20 / 66$ to $2 / 7 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-464$ | G | $2 / 7 / 66$ | 0.500 | 0.350 |  |
| $66-3270$ | AS | $6 / 1 / 66$ | 0.146 | 0.147 | $\checkmark$ |
| $66-3270$ | G | $6 / 1 / 66$ | NR | NR |  |

* G means gross alpha counting; AS means alpha spectrometry.

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

Modeling:
CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake $(\mathrm{pCi})$ | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 210,000 | $65 / 0.65$ |
| LUDEP | 591,000 | $42 / 0.42$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $9.4 \mathrm{E}+00 / 9.4 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $2.3 \mathrm{E}+00 / 2.3 \mathrm{E}-02$ |
| Breast | $3.1 \mathrm{E}-04 / 3.1 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $4.7 \mathrm{E}-05 / 4.7 \mathrm{E}-05$ |
| Red Marrow | $5.1 \mathrm{E}+01 / 5.1 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $6.1 \mathrm{E}+00 / 6.1 \mathrm{E}-02$ |
| Lung | $2.4 \mathrm{E}+02 / 2.4 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $2.9 \mathrm{E}+01 / 2.9 \mathrm{E}-01$ |
| Thyroid | $2.9 \mathrm{E}-04 / 2.9 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $8.8 \mathrm{E}-06 / 8.8 \mathrm{E}-06$ |
| Bone Surface | $6.5 \mathrm{E}+02 / 6.5 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $2.0 \mathrm{E}+01 / 2.0 \mathrm{E}-01$ |
| Liver | $1.2 \mathrm{E}+02 / 1.2 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $7.0 \mathrm{E}+007 / .0 \mathrm{E}-02$ |
| Other | $1.1 \mathrm{E}+01 / 1.1 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $6.7 \mathrm{E}-0167 \mathrm{E}-03$ |
| Lower Large Intestine | $2.4 \mathrm{E}-02 / 2.4 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $1.4 \mathrm{E}-03 / 1.4 \mathrm{E}-05$ |
| Upper Large Intestine | $8.0 \mathrm{E}-03 / 8.0 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $4.8 \mathrm{E}-04 / 4.8 \mathrm{E}-06$ |
| Small Intestine | $1.6 \mathrm{E}-03 / 1.6 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $9.8 \mathrm{E}-07 / 9.8 \mathrm{E}-07$ |
| Effective Dose Equivalent |  | $6.5 \mathrm{E}+01 / 6.5 \mathrm{E}-01$ |  |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. The sample analyzed by gross alpha counting only was not included in the analysis since it was an on-site sample with a result $>0.1 \mathrm{pCi}$, leading to a suspicion of sample contamination. The result was fit using CINDY and the Jones excretion model, to estimate an intake $(210,000 \mathrm{pCi})$, organ doses, and a CEDE ( $65 \mathrm{rem} / 0.65 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $591,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of 42 rem ( 0.42 Sv ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $210,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 65 rem ( 0.65 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is more than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not normally associated with these dose levels. However, follow-up urine sampling should be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$


ONE TIME FORM. OBSOLETE AFTER 30 JUN 66. (MCGSCPF, RAG, 4 APR 66)

## FEB 101966



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## Internal Dosimetry Evaluation Form

| NAME: ${ }^{(10)}$ (6) |  |  | ATE OR PERIOD: <br> 2/26/66, onsite |  |
| :---: | :---: | :---: | :---: | :---: |
|  | $\square$ Injection $\square$ Absorption $\square$ Not applicable | INTAKE DATE OR PERIOD: 2/5/66 through 2/26/66, onsite 2/15/66 |  |  |
| SUMMARY OF EXPOSURE CONDITIONS: <br> Radionuclides/Respiratory Class/Particle Size: ${ }^{239} \mathrm{Pu} / 100 \%$ Class Y/1 $\mu \mathrm{m}$ AMAD <br> Date or Period of Evaluated Data: 1 sample, 2/26/66 <br> Duration of Exposure: Unknown <br> Location of Exposure: Camp Wilson, near Palomares, Spain |  |  |  |  |
| EVALUATION DATA: |  |  |  |  |
| Medical Treatment: <br> Skin Decontamination: <br> Decorporation: <br> Catharsis: <br> Surgical excision: | $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ | 《 No Date: <br> $\boxtimes$ No <br> Agent  <br> Q  <br> No Agent: <br> No Date: |  | Date: <br> Date: |

## EVALUATION METHODOLOGY:

Assumptions: Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 15 / 66$
Code/Model used for: Intake Estimate: CINDY, Ver. 1.4/JONES
Dose Estimate: CINDY, Ver. 1.4/ICRP 30, Part 4, General Systemic Model
RESULTS SUMMARY
Estimated Intake Activity ( pCi ): 7700
50 YR CEDE (rem) : 2.4 (0.024 Sv)

| Organ Dose Equivalent Summary | 50 YR CDE (rem/Sv) |
| :---: | :---: |
| Bone Surface | $24 / 0.24$ |
| Lung | $8.8 / 0.088$ |
| Liver | $4.3 / 0.043$ |
| Red Marrow | $1.9 / 0.019$ |
| Other | $0.41 / 0.0041$ |
| Testes | $0.34 / 0.0034$ |


| DOSE ASSESSOR: <br> Signature: | DATE: | PEER REVIEWER: <br> Signature: $\qquad$ | DATE: |
| :---: | :---: | :---: | :---: |
|  |  |  | 8 |
| Print Name: |  | Print Name: |  |
| SSN: |  | SSN: |  |

## RECOMMENDATIONS:

Additional Bioassay Required
Suggested Sampling Frequency:Urinalysis $\square$ Fecal $\square$ In Vivo

Work Restrictions: N/A N/A

[^0]Internal Dosimetry Case Narrative

## Identification:

## Name:

SSN:

## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 15 / 66$. The date is the midpoint of the period on station from $2 / 5 / 66$ to $2 / 26 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1430$ | AS | $2 / 26 / 66$ | NR | NR |  |
| $66-1430$ | G | $2 / 26 / 66$ | ND | ND | $\checkmark$ |
| *G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 7,700 | $2.4 / 0.024$ |
| LUDEP | 38,300 | $2.7 / 0.027$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> $($ rem/Sv |
| :--- | :---: | :---: | :---: |
| Testes | $3.4 \mathrm{E}-01 / 3.4 \mathrm{E}-03$ | $2.5 \mathrm{E}-01$ | $8.6 \mathrm{E}-02 / 8.6 \mathrm{E}-04$ |
| Breast | $1.1 \mathrm{E}-05 / 1.1 \mathrm{E}-07$ | $1.5 \mathrm{E}-01$ | $1.7 \mathrm{E}-06 / 1.7 \mathrm{E}-08$ |
| Red Marrow | $1.9 \mathrm{E}+00 / 1.9 \mathrm{E}-02$ | $1.2 \mathrm{E}-01$ | $2.2 \mathrm{E}-01 / 2.2 \mathrm{E}-03$ |
| Lung | $8.8 \mathrm{E}+00 / 8.8 \mathrm{E}-02$ | $1.2 \mathrm{E}-01$ | $1.1 \mathrm{E}+00 / 1.1 \mathrm{E}-02$ |
| Thyroid | $1.1 \mathrm{E}-05 / 1.1 \mathrm{E}-07$ | $3.0 \mathrm{E}-02$ | $3.2 \mathrm{E}-07 / 3.2 \mathrm{E}-09$ |
| Bone Surface | $2.4 \mathrm{E}+01 / 2.4 \mathrm{E}-01$ | $3.0 \mathrm{E}-02$ | $7.2 \mathrm{E}-01 / 7.2 \mathrm{E}-03$ |
| Liver | $4.3 \mathrm{E}+00 / 4.3 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $2.6 \mathrm{E}-01 / 2.6 \mathrm{E}-03$ |
| Other | $4.1 \mathrm{E}-01 / 4.1 \mathrm{E}-03$ | $6.0 \mathrm{E}-02$ | $2.4 \mathrm{E}-02 / 2.4 \mathrm{E}-04$ |
| Lower Large Intestine | $8.7 \mathrm{E}-04 / 8.7 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $5.2 \mathrm{E}-05 / 5.2 \mathrm{E}-07$ |
| Upper Large Intestine | $2.9 \mathrm{E}-04 / 2.9 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $1.8 \mathrm{E}-051.1 .8 \mathrm{E}-07$ |
| Small Intestine | $6.0 \mathrm{E}-05 / 6.0 \mathrm{E}-07$ | $6.0 \mathrm{E}-02$ | $3.6 \mathrm{E}-06 / 3.6 \mathrm{E}-08$ |
| Effective Dose Equivalent |  |  | $2.4 \mathrm{E}+00 / 2.4 \mathrm{E}-02$ |

One urine sample was analyzed by gross alpha counting and alpha spectrometry. The alpha spectrometry analysis was not included in the modeling since no result was reported. The gross alpha counting result was reported as No Detectable Activity. A value of 0.009 pCi was used to represent this outcome. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $7,700 \mathrm{pCi}$ ), organ doses, and a CEDE ( $2.4 \mathrm{rem} / 0.024 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $38,300 \mathrm{pCi}$ and a CEDE (ICRP-60) of 2.7 rem ( 0.027 Sv ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $7,700 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 2.4 rem ( 0.024 Sv ). That dose is less than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level ( 0.100 rem ) for members of the public. It is far less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$



Internal Dosimetry Evaluation Form

| NAME: ${ }^{(15)}$ (6) | SSN: ${ }^{(10)}{ }^{(6)}$ |  |  |  |
| :---: | :---: | :---: | :---: | :---: |
|  | Injection Absorption Not applicable | INTAKE DATE OR PERIOD: 1/18/66 through $1 / 31 / 66$, onsite 1/24/66 |  |  |
| SUMMARY OF EXPOSURE CONDITIONS: <br> Radionuclides/Respiratory Class/Particle Size: ${ }^{239} \mathrm{Pu} / 100 \%$ Class Y/1 $\mu \mathrm{m}$ AMAD <br> Date or Period of Evaluated Data: 4 samples, 1/21/66 through 6/2/66 <br> Duration of Exposure: Unknown <br> Location of Exposure: Camp Wilson, near Palomares, Spain |  |  |  |  |
| EVALUATION DATA: <br> Air Sampling Health Physics Survey Data Bioassay - Urinalysis Fecal $\begin{array}{lll}\square \text { Attached } & \square \text { In Process } & \boxtimes \text { Unavailable } \\ \square \text { Attached } & \square \text { In Process } & \text { Q Unavailable } \\ \boxtimes \text { Attached } & \square \text { In Process } & \square \text { Unavailable } \\ \square \text { Attached } & \square \text { In Process } & \text { Q Unavailable } \\ \square \text { Attached } & \square \text { In Process } & \text { U Unavailable } \\ \square \text { Attached } & \square \text { In Process } & \boxed{\text { Unavailable }}\end{array}$ |  |  |  |  |
| Medical Treatment: <br> Skin Decontamination: <br> Decorporation: <br> Catharsis: <br> Surgical excision: | $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ | $\boxtimes$ No Date: <br> $\boxtimes$ No Agent <br> 区 No Agent <br> 区 No Date: | $\qquad$ | Date: <br> Date: |

EVALUATION METHODOLOGY:

| Assumptions: Acute inhalation intake of |  |
| :--- | :--- |
| Code/Model used for: |  |
|  | Intake Estimate: CINDY, Ver. 1.4/JONES |
|  | Dose Estimate: CINDY, Ver. 1.4/ICRP 30, Part 4, General Systemic Model |

RESULTS SUMMARY
Estimated Intake Activity (pCi): 240,000
50 YR CEDE (rem) : 74 ( 0.74 Sv )

| Organ Dose Equivalent Summary | 50 YR CDE (rem/Sv) |
| :--- | :---: |
| Bone Surface | $750 / 7.5$ |
| Lung | $270 / 2.7$ |
| Liver | $130 / 1.3$ |
| Red Marrow | $58 / 0.58$ |
| Other | $13 / 0.13$ |
| Testes | $11 / 0.11$ |


| DOSE ASSESSOR: | DATE: | PEER REVIEWER: <br> Signature: $\qquad$ | DATE: |
| :---: | :---: | :---: | :---: |
| Signature: |  |  |  |
| Print Name: |  | Print Name: |  |
| SSN: |  | SSN: |  |

## RECOMMENDATIONS:

Additional Bioassay Required
Suggested Sampling Frequency:
Work Restrictions:
$\square$ UrinalysisFecalIn Vivo

[^1]
## Internal Dosimetry Case Narrative

## Identification:

## Name:

SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

Other Information:
None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $1 / 24 / 66$. The date is the midpoint of the period on station from $1 / 18 / 66$ to $1 / 31 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-267$ | G | $1 / 21 / 66$ | NR | NR |  |
| $66-409$ | G | $2 / 2 / 66$ | 1.02 | 0.750 |  |
| $66-568$ | G | $2 / 4 / 66$ | 1.86 | 0.089 |  |
| $66-3407$ | AS | $6 / 2 / 66$ | 0.168 | 0.089 | $\checkmark$ |
| $66-3407$ | G | $6 / 2 / 66$ | NR | NR |  |

[^2]Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CINDY | 240,000 | $74 / 0.74$ |
| LUDEP | 680,000 | $48 / 0.48$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem $/ S v)$ | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $1.1 \mathrm{E}+01 / 1.1 \mathrm{E}-01$ | $2.5 \mathrm{E}-01$ | $2.7 \mathrm{E}+00 / 2.7 \mathrm{E}-02$ |
| Breast | $3.6 \mathrm{E}-04 / 3.6 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $5.3 \mathrm{E}-05 / 5.3 \mathrm{E}-07$ |
| Red Marrow | $5.8 \mathrm{E}+01 / 5.8 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $6.9 \mathrm{E}+00 / 6.9 \mathrm{E}-02$ |
| Lung | $2.7 \mathrm{E}+02 / 2.7 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $3.3 \mathrm{E}+01 / 3.3 \mathrm{E}-01$ |
| Thyroid | $3.3 \mathrm{E}-04 / 3.3 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $1.0 \mathrm{E}-05 / 1.0 \mathrm{E}-07$ |
| Bone Surface | $7.5 \mathrm{E}+02 / 7.5 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $2.2 \mathrm{E}+01 / 2.2 \mathrm{E}-01$ |
| Liver | $1.3 \mathrm{E}+02 / 1.3 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $8.0 \mathrm{E}+00 / 8.0 \mathrm{E}-02$ |
| Other | $1.3 \mathrm{E}+01 / 3 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $7.6 \mathrm{E}+00 / 7.6 \mathrm{E}-02$ |
| Lower Large Intestine | $2.7 \mathrm{E}-02 / 2.7 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $1.6 \mathrm{E}-03 / 1.6 \mathrm{E}-05$ |
| Upper Large Intestine | $9.1 \mathrm{E}-03 / 9.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $5.5 \mathrm{E}-04 / 5.5 \mathrm{E}-06$ |
| Small Intestine | $1.9 \mathrm{E}-03 / 1.9 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $1.1 \mathrm{E}-04 / 1.1 \mathrm{E}-06$ |
| Effective Dose Equivalent |  | $7.4 \mathrm{E}+01 / 7.4 \mathrm{E}-01$ |  |

Three urine samples were analyzed by gross alpha counting only, and the fourth was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. One of the samples analyzed by gross alpha counting only was not included in the analysis since no result was reported. The other gross alpha results were excluded from the analysis because they did not fit the expected pattern of plutonium excretion and because they may
have been contaminated during sample collection on the site . have been contaminated during sample collection on the site. The remaining result was fit using CINDY and the Jones excretion model, to estimate an intake ( $240,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( 74 rem/ 0.74 Sv ; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $680,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $48 \mathrm{rem}(0.48 \mathrm{~Sv})$.

When all results were included in a separate evaluation, CINDY produced estimated intake and CEDE of $970,000 \mathrm{pCi}$ and $300 \mathrm{rem}(3.0 \mathrm{~Sv}$ ). LUDEP estimated intake and CEDE at $4,100,000 \mathrm{pCi}$ and 290 rem (2.9 Sv).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $240,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $74 \mathrm{rem} \mathrm{( } 0.74$ Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is more than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). These doses are significant, although they are based in part, on samples collected on-site and potentially contaminated.
However, follow-up urine sampling should be considered However, follow-up urine sampling should be considered to provide additional assessment of the
exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

[^3]


$\therefore . \therefore 86$ TAC hose. Apo NY 09012
$\qquad$
$409 \# 4$

SN.

/ BOTTLE $\qquad$
$\qquad$
$\qquad$
$\qquad$
AMT USED 200 Mc NITRIC ACID 50 ML


Internal Dosimetry Evaluation Form

| NAME：${ }^{(1))^{(6)}}$ |  |  | SSN：［（b）（6） |  |
| :---: | :---: | :---: | :---: | :---: |
|  | $\square$ Injection $\square$ Absorption $\square$ Not applicable | INTAKE D <br> 1／18／66 throu <br> 2／8／66 | TE OR PERIOD： h $3 / 1 / 66$ ，onsite |  |
| SUMIMARY OF EXPOSURE CONDITIONS： <br> Radionuclides／Respiratory Class／Particle Size：${ }^{239} \mathrm{Pu} / 100 \%$ Class Y／1 $\mu \mathrm{m}$ AMAD <br> Date or Period of Evaluated Data： 1 sample，3／1／66 <br> Duration of Exposure：Unknown <br> Location of Exposure：Camp Wilson，near Palomares，Spain |  |  |  |  |
| EVALUATION DATA： <br> Air Sampling Health Physics Survey Data Bioassay－Urinalysis Fecal Nasal Smears In Vivo | $\square$ Attached $\square$ Attached $\boxtimes$ Attached $\square$ Attached $\square$ Attached $\square$ Attached | In Process In Process In Process In Process In Process In Process | 区 Unavailable <br> 区 Unavailable <br> $\square$ Unavailable <br> 区 Unavailable <br> 区 Unavailable <br> 区 Unavailable |  |
| Medical Treatment： <br> Skin Decontamination： <br> Decorporation： <br> Catharsis： <br> Surgical excision： | $\square$ Yes $\square$ Yes $\square$ Yes $\square$ Yes | 区 No Date： <br> 区 No Agent： <br> 区 No Agent： <br> 区 No Date： |  | Date： Date： |

EVALUATION METHODOLOGY：
Assumptions：Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on 2／8／66
$\begin{array}{ll}\text { Code／Model used for：} & \text { Intake Estimate：CINDY，Ver．1．4／JONES } \\ & \text { Dose Estimate：} \\ & \text { CINDY，Ver．1．4／ICRP 30，Part 4，General Systemic Model }\end{array}$
RESULTS SUMIMARY
Estimated Intake Activity（pCi）： 270000
50 YR CEDE（rem）： 83 （ 0.83 Sv ）


## RECOMMIENDATIONS：

| Additional Bioassay Required |  |  |  |  |
| :--- | :--- | :--- | :--- | :--- |
| Suggested Sampling Frequency： |  |  |  |  |
| Work Restrictions： | N／A | $\square$ Urinalysis | $\square$ Fecal | $\square$ In Vivo |

Internal Dosimetry Case Narrative

## Identification:



## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; 100\% Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on 2/8/66. The date is the midpoint of the period on station from 1/18/66 to 3/1/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1849$ | AS | $3 / 1 / 66$ | 0.259 | 0.151 | $\checkmark$ |
| $66-1849$ | $G$ | $3 / 1 / 66$ | 0.924 | 0.249 |  |
| G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv}$ ) |
| :--- | :---: | :---: |
| CINDY | 270,000 | $83 / 0.83$ |
| LUDEP | $1,110,000$ | $78 / 0.78$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> $($ rem $/ S v)$ |
| :--- | :---: | :---: | :---: |
| Testes | $1.2 \mathrm{E}+01 / 1.2 \mathrm{E}-01$ | $2.5 \mathrm{E}-01$ | $3.0 \mathrm{E}+00 / 3.0 \mathrm{E}-02$ |
| Breast | $4.0 \mathrm{E}-04 / 4.0 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $6.0 \mathrm{E}-05 / 6.0 \mathrm{E}-07$ |
| Red Marrow | $6.5 \mathrm{E}+01 / 6.5 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $7.8 \mathrm{E}+00 / 7.8 \mathrm{E}-02$ |
| Lung | $3.1 \mathrm{E}+02 / 3.1 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $3.7 \mathrm{E}+01 / 3.7 \mathrm{E}-01$ |
| Thyroid | $3.8 \mathrm{E}-04 / 3.8 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $1.1 \mathrm{E}-05 / 1.1 \mathrm{E}-07$ |
| Bone Surface | $8.4 \mathrm{E}+02 / 8.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $2.5 \mathrm{E}+01 / 2.5 \mathrm{E}-01$ |
| Liver | $1.5 \mathrm{E}+02 / 1.5 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $9.0 \mathrm{E}+00 / 9.0 \mathrm{E}-02$ |
| Other | $1.4 \mathrm{E}+01 / 1.4 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $8.6 \mathrm{E}-01 / 8.6 \mathrm{E}-03$ |
| Lower Large Intestine | $3.1 \mathrm{E}-02 / 3.1 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $1.8 \mathrm{E}-03 / 1.8 \mathrm{E}-05$ |
| Upper Large Intestine | $1.0 \mathrm{E}-02 / 1.0 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $6.2 \mathrm{E}-04 / 6.2 \mathrm{E}-06$ |
| Small Intestine | $2.1 \mathrm{E}-03 / 2.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $1.3 \mathrm{E}-04 / 1.3 \mathrm{E}-06$ |
| Effective Dose Equivalent |  |  | $8.3 \mathrm{E}+01 / 8.3 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting and alpha spectrometry. The gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $270,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $83 \mathrm{rem} / 0.83 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $1,110,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $78 \mathrm{rem}(0.78 \mathrm{~Sv}$ ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $270,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 83 rem ( 0.83 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level ( 0.100 rem ) for members of the public. It is more than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). These dose levels are significant although base on only one data point. However, follow-up urine sampling should be considered to provide additional assessment of the exposure.

Prepared By:
Name: $\qquad$

Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

(b) (6)

Internal Dosimetry Evaluation Form


| RECOMMENDATIONS: |  |  |  |  |
| :--- | :--- | :--- | :--- | :--- |
| Additional Bioassay Required <br> Suggested Sampling Frequency: <br> Work Restrictions: | N/A |  | $\square$ Urinalysis | $\square$ Fecal |$\quad \square$ In Vivo

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Internal Dosimetry Case Narrative

## Identification:

## Name:

SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; 100\% Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $1 / 31 / 66$. The date is the midpoint of the period on station from 1/23/66 to 2/9/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally - windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-814$ | $G$ | $1 / 29 / 66$ | $1.31(12-\mathrm{hr})$ | 0.510 |  |
| $66-3401$ | AS | $6 / 2 / 66$ | 0.099 | 0.047 | $\checkmark$ |
| $66-3401$ | G | $6 / 2 / 66$ | NR | NR |  |
| G means gross alpha counting; AS means alpha spectrometry; |  | NR means no result recorded. |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

Modeling:
CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CINDY | 140,000 | $43 / 0.43$ |
| LUDEP | 402,000 | $28 / 0.28$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> (rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $6.2 \mathrm{E}+00 / 6.2 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.6 \mathrm{E}+00 / 1.6 \mathrm{E}-02$ |
| Breast | $2.1 \mathrm{E}-04 / 2.1 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $3.1 \mathrm{E}-05 / 3.1 \mathrm{E}-07$ |
| Red Marrow | $3.4 \mathrm{E}+01 / 3.4 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $4.0 \mathrm{E}+00 / 4.0 \mathrm{E}-02$ |
| Lung | $1.6 \mathrm{E}+02 / 1.6 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.9 \mathrm{E}+01 / 1.9 \mathrm{E}-01$ |
| Thyroid | $2.0 \mathrm{E}-04 / 2.0 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $5.9 \mathrm{E}-06 / 5.9 \mathrm{E}-08$ |
| Bone Surface | $4.4 \mathrm{E}+02 / 4.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.3 \mathrm{E}+01 / 1.3 \mathrm{E}-01$ |
| Liver | $7.8 \mathrm{E}+01 / 7.8 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $4.7 \mathrm{E}+00 / 4.7 \mathrm{E}-02$ |
| Other | $7.4 \mathrm{E}+00 / 7.4 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $4.4 \mathrm{E}-01 / 4.4 \mathrm{E}-03$ |
| Lower Large Intestine | $1.6 \mathrm{E}-02 / 1.6 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $9.5 \mathrm{E}-04 / 9.5 \mathrm{E}-06$ |
| Upper Large Intestine | $5.3 \mathrm{E}-03 / 5.3 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $3.2 \mathrm{E}-04 / 3.2 \mathrm{E}-06$ |
| Small Intestine | $1.1 \mathrm{E}-03 / 1.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $6.5 \mathrm{E}-05 / 6.5 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $4.3 \mathrm{E}+01 / 4.3 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. The sample analyzed by gross alpha counting only was not included in the analysis since it was an on-site sample with a result $>0.1 \mathrm{pCi}$, leading to a suspicion of sample contamination. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $140,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $43 \mathrm{rem} / 0.43 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $402,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $28 \mathrm{rem}(0.28 \mathrm{~Sv})$.

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $140,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 43 rem ( 0.43 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$





## Internal Dosimetry Evaluation Form



## EVALUATION METHODOLOGY:

Assumptions: Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on $1 / 23 / 66$

| Code/Model used for: | Intake Estimate: CINDY, Ver. 1.4/JONES |
| :--- | :--- |
|  | Dose Estimate: CINDY, Ver. 1.4/ICRP 30, Part 4, General Systemic Model |

RESULTS SUMMARY
Estimated Intake Activity (pCi): 110000
50 YR CEDE (rem) : 34 ( 0.34 Sv )

| Organ Dose Equivalent Summary | 50 YR CDE (rem/Sv) |  |  |
| :--- | :---: | :---: | :---: |
| Bone Surface | $340 / 3.4$ |  |  |
| Lung | $130 / 1.3$ |  |  |
| Liver | $61 / 0.61$ |  |  |
| Red Marrow | $26 / 0.26$ |  |  |
| Other | $5.8 / 0.058$ |  |  |
| Testes | $4.9 / 0.049$ |  |  |
| DOSE ASSESSOR: |  | PEER REVIIEWER: | DATE: |

Signature: $\qquad$ Signature: $\qquad$
Print Name: $\qquad$
SSN:
Print Name: $\qquad$
SSN:


# Internal Dosimetry Case Narrative 

## Identification:

Name:
SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $1 / 23 / 66$. The date is the midpoint of the period on station from $1 / 18 / 66$ to $1 / 29 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> $(\mathrm{pCi} /$ day $)$ | Error <br> $(\mathrm{pCi} /$ day $)$ | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1209$ | G | $1 / 29 / 66$ | 6.20 | 2.18 |  |
| $66-3400$ | AS | $5 / 30 / 66$ | 0.078 | 0.057 | $\checkmark$ |
| $66-3400$ | G | $5 / 30 / 66$ | NR | NR |  |
| * G means gross alpha counting; AS means alpha spectrometry; | NR means no result reported. |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake $(\mathrm{pCi})$ | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 110,000 | $34 / 0.34$ |
| LUDEP | 316,000 | $22 / 0.22$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $(\mathrm{rem} / \mathrm{Sv})$ | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $4.9 \mathrm{E}+00 / 4.9 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.2 \mathrm{E}+00 / 1.2 \mathrm{E}-02$ |
| Breast | $1.6 \mathrm{E}-04 / 1.6 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $2.4 \mathrm{E}-05 / 2.4 \mathrm{E}-07$ |
| Red Marrow | $2.6 \mathrm{E}+01 / 2.6 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $3.2 \mathrm{E}+00 / 3.2 \mathrm{E}-02$ |
| Lung | $1.3 \mathrm{E}+02 / 1.3 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.5 \mathrm{E}+01 / 1.5 \mathrm{E}-01$ |
| Thyroid | $1.5 \mathrm{E}-04 / 1.5 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $4.6 \mathrm{E}-06 / 4.6 \mathrm{E}-08$ |
| Bone Surface | $3.4 \mathrm{E}+02 / 3.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.0 \mathrm{E}+01 / 1.0 \mathrm{E}-01$ |
| Liver | $6.1 \mathrm{E}+01 / 6.1 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $3.7 \mathrm{E}+00 / 3.7 \mathrm{E}-02$ |
| Other | $5.8 \mathrm{E}+00 / 5.8 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $3.5 \mathrm{E}-01 / 3.5 \mathrm{E}-03$ |
| Lower Large Intestine | $1.2 \mathrm{E}-02 / 1.2 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $7.5 \mathrm{E}-04 / 7.5 \mathrm{E}-06$ |
| Upper Large Intestine | $4.2 \mathrm{E}-03 / 4.2 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $2.5 \mathrm{E}-04 / 2.5 \mathrm{E}-06$ |
| Small Intestine | $8.6 \mathrm{E}-04 / 8.6 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $5.1 \mathrm{E}-05 / 5.1 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $3.4 \mathrm{E}+01 / 3.4 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. The sample analyzed by gross alpha counting only was not included in the analysis since it was an on-site sample with a result $>0.1 \mathrm{pCi}$, leading to a suspicion of sample contamination. The result was fit using CINDY and the Jones excretion model, to estimate an intake $(110,000 \mathrm{pCi})$, organ doses, and a CEDE ( $34 \mathrm{rem} / 0.34 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $316,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of 22 rem ( 0.22 Sv ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $110,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 34 rem ( 0.34 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level $(0.100 \mathrm{rem})$ for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$
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## Internal Dosimetry Evaluation Form



## RECOMMENDATIONS:

Additional Bioassay Required
Suggested Sampling Frequency:
Work Restrictions:Urinalysis $\square$ Fecal $\square$ In Vivo

# Internal Dosimetry Case Narrative 

## Identification:

Name:
SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $1 / 31 / 66$. The date is the midpoint of the period on station from 1/18/66 to 2/14/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1149$ | G | $2 / 5 / 66$ | ND | ND | $\checkmark$ |
| $66-2811$ | AS | $4 / 8 / 66$ | 0.076 | 0.007 | $\checkmark$ |
| $66-2871$ | G | $4 / 8 / 66$ | 1.45 | 0.310 |  |
| * G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

A nasal swipe (sample \#66-1334) was recorded for result was available.

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake $(\mathrm{pCi})$ | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 28,000 | $8.6 / 0.086$ |
| LUDEP | 42,400 | $3.0 / 0.03$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $(\mathrm{rem} / \mathrm{Sv})$ | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: | :---: |
| Testes | $1.2 \mathrm{E}+00 / 1.2 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $3.1 \mathrm{E}-01 / 3.1 \mathrm{E}-03$ |
| Breast | $4.1 \mathrm{E}-05 / 4.1 \mathrm{E}-07$ | $1.5 \mathrm{E}-01$ | $6.2 \mathrm{E}-06 / 6.2 \mathrm{E}-08$ |
| Red Marrow | $6.7 \mathrm{E}+00 / 6.7 \mathrm{E}-02$ | $1.2 \mathrm{E}-01$ | $8.1 \mathrm{E}-01 / 8.1 \mathrm{E}-03$ |
| Lung | $3.2 \mathrm{E}+01 / 3.2 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $3.8 \mathrm{E}+00 / 3.8 \mathrm{E}-02$ |
| Thyroid | $3.9 \mathrm{E}-05 / 3.9 \mathrm{E}-07$ | $3.0 \mathrm{E}-02$ | $1.2 \mathrm{E}-06 / 1.2 \mathrm{E}-08$ |
| Bone Surface | $8.7 \mathrm{E}+01 / 8.7 \mathrm{E}-01$ | $3.0 \mathrm{E}-02$ | $2.6 \mathrm{E}+00 / 2.6 \mathrm{E}-02$ |
| Liver | $1.6 \mathrm{E}+01 / 1.6 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $9.4 \mathrm{E}-01 / 9.4 \mathrm{E}-03$ |
| Other | $1.5 \mathrm{E}+00 / 1.5 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $8.9 \mathrm{E}-02 / 8.9 \mathrm{E}-04$ |
| Lower Large Intestine | $3.2 \mathrm{E}-03 / 3.2 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $1.9 \mathrm{E}-04 / 1.9 \mathrm{E}-06$ |
| Upper Large Intestine | $1.1 \mathrm{E}-03 / 1.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $6.4 \mathrm{E}-05 / 6.4 \mathrm{E}-07$ |
| Small Intestine | $2.2 \mathrm{E}-04 / 2.2 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $1.3 \mathrm{E}-05 / 1.3 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $8.6 \mathrm{E}+00 / 8.6 \mathrm{E}-02$ |

One urine sample was analyzed by gross alpha counting, and the other was analyzed by both gross alpha counting and alpha spectrometry. The gross alpha analysis for the second sample was not included in the modeling since an alpha spectrometry result was available for the same sample. The sample result that was analyzed using gross alpha counting only was reported as No Detectable Activity. A value of 0.009 pCi was used to represent this outcome. The results were fit using CINDY and the Jones excretion model, to estimate an intake ( $28,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $8.6 \mathrm{rem} / 0.086 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $42,400 \mathrm{pCi}$ and a CEDE (ICRP-60) of 3.0 rem (0.03 Sv).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $28,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $8.6 \mathrm{rem}(0.086$ Sv). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is less than half the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$






## Internal Dosimetry Evaluation Form

| NAME: ${ }^{(1)}$ (G) |  |  |  | SSN: (b) (6) |  |
| :---: | :---: | :---: | :---: | :---: | :---: |
| MODE OF INTAKE: 区 Inhalation $\square$ Ingestion $\square$ Unknown | Injection Absorption Not applicable | INTAKE DATE OR PERIOD: 1/18/66 through 4/8/66, onsite 2/27/66 |  |  |  |
| SUMMARY OF EXPOSURE CONDITIONS: <br> Radionuclides/Respiratory Class/Particle Size: ${ }^{239} \mathrm{Pu} / 100 \%$ Class Y/1 $\mu \mathrm{m}$ AMAD <br> Date or Period of Evaluated Data: 1 sample, 4/8/66 <br> Duration of Exposure: Unknown <br> Location of Exposure: Camp Wilson, near Palomares, Spain |  |  |  |  |  |
| EVALUATION DATA: |  |  |  |  |  |
| Medical Treatment: <br> Skin Decontamination: <br> Decorporation: <br> Catharsis: <br> Surgical excision: | $\square$ Yes $\square$ Yes $\square$ Yes $\square$ Yes | $\boxtimes$ No $\boxtimes$ No $\boxtimes$ No $\boxtimes$ No | Date: <br> Agent: <br> Agent: <br> Date: |  | Date: Date: |

## EVALUATION METHODOLOGY:

Assumptions: Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 27 / 66$
Code/Model used for: Intake Estimate: CINDY, Ver. 1.4/JONES
Dose Estimate: CINDY, Ver. 1.4/ICRP 30, Part 4, General Systemic Model
RESULTS SUMIMARY
Estimated Intake Activity (pCi): 95000
50 YR CEDE (rem) : 29 ( 0.29 Sv )
Organ Dose Equivalent Summary 50 YR CDE (rem/Sv)
Bone Surface

300/3
Lung
110/1.1
Liver
53/0.53
Red Marrow
23/0.23
Other 5/0.05
Testes
4.2/0.042

| DOSE ASSESSOR: | DATE: | PEER REVIEWER: <br> Signature: | DATE: |
| :---: | :---: | :---: | :---: |
| Signature: |  |  |  |
| Print Name: |  | Print Name: | - |
| SSN: |  | SSN: |  |

## RECOMMENDATIONS:

| Additional Bioassay Required |  |
| :--- | :--- | :--- | :--- | :--- |
| Suggested Sampling Frequency: |  |
| Work Restrictions: | N/A |$\quad \square$ Urinalysis $\quad \square$ Fecal $\quad \square$ In Vivo

## Internal Dosimetry Case Narrative

## Identification:

Name:
SSN:

## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $2 / 27 / 66$. The date is the midpoint of the period on station from 1/18/66 to 4/8/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-2861$ | AS | $4 / 8 / 66$ | 0.074 | 0.011 | $\checkmark$ |
| $66-2861$ | G | $4 / 8 / 66$ | 1.29 | 0.290 |  |
| * G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CINDY | 95,000 | $29 / 0.29$ |
| LUDEP | 316,000 | $22 / 0.22$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> (rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $4.2 \mathrm{E}+00 / 4.2 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.1 \mathrm{E}+00 / 1.1 \mathrm{E}-02$ |
| Breast | $1.4 \mathrm{E}-04 / 1.4 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $2.1 \mathrm{E}-05 / 2.1 \mathrm{E}-07$ |
| Red Marrow | $2.3 \mathrm{E}+01 / 2.3 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $2.7 \mathrm{E}+00 / 2.7 \mathrm{E}-02$ |
| Lung | $1.1 \mathrm{E}+02 / 1.1 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.3 \mathrm{E}+01 / 1.3 \mathrm{E}-01$ |
| Thyroid | $1.3 \mathrm{E}-04 / 1.3 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $4.0 \mathrm{E}-06 / 4.0 \mathrm{E}-08$ |
| Bone Surface | $3.0 \mathrm{E}+02 / 3.0 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $8.9 \mathrm{E}+00 / 8.9 \mathrm{E}-02$ |
| Liver | $5.3 \mathrm{E}+01 / 5.3 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $3.2 \mathrm{E}+00 / 3.2 \mathrm{E}-02$ |
| Other | $5.0 \mathrm{E}+00 / 5.0 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $3.0 \mathrm{E}-01 / 3.0 \mathrm{E}-03$ |
| Lower Large Intestine | $1.1 \mathrm{E}-02 / 1.1 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $6.5 \mathrm{E}-04 / 6.5 \mathrm{E}-06$ |
| Upper Large Intestine | $3.6 \mathrm{E}-03 / 3.6 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $2.2 \mathrm{E}-04 / 2.2 \mathrm{E}-06$ |
| Small Intestine | $7.4 \mathrm{E}-04 / 7.4 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $4.4 \mathrm{E}-05 / 4.4 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $2.9 \mathrm{E}+01 / 2.9 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting and alpha spectrometry. The gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $95,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $29 \mathrm{rem} / 0.29 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $316,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of 22 rem ( 0.22 Sv ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $95,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $29 \mathrm{rem}(0.29$ Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level ( 0.100 rem ) for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level.

Prepared By:
Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$




Internal Dosimetry Evaluation Form


RECOMIMIENDATIONS:

| Additional Bioassay Required  <br> Suggested Sampling Frequency:  <br> Work Restrictions: N/A | $\square$ Urinalysis | $\square$ Fecal | $\square$ In Vivo |
| :--- | :--- | :--- | :--- | :--- |
|  |  |  |  |

## Internal Dosimetry Case Narrative

## Identification:



## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of $\mathrm{Pu}-239 ; 100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 9 / 66$. The date is the midpoint of the period on station from $1 / 18 / 66$ to 3/4/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-2869$ | AS | $4 / 6 / 66$ | 0.224 | 0.007 | $\checkmark$ |
| *-2869 | G | $4 / 6 / 66$ | 1.75 | 0.340 |  |
| * G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 310,000 | $95 / 0.95$ |
| LUDEP | 950,000 | $67 / 0.67$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem $/$ Sv $)$ | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $1.4 \mathrm{E}+01 / 1.4 \mathrm{E}-01$ | $2.5 \mathrm{E}-01$ | $3.5 \mathrm{E}+00 / 3.5 \mathrm{E}-02$ |
| Breast | $4.6 \mathrm{E}-04 / 4.6 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $6.9 \mathrm{E}-05 / 6.9 \mathrm{E}-07$ |
| Red Marrow | $7.5 \mathrm{E}+01 / 7.5 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $9.0 \mathrm{E}+00 / 9.0 \mathrm{E}-02$ |
| Lung | $3.5 \mathrm{E}+02 / 3.5 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $4.2 \mathrm{E}+01 / 4.2 \mathrm{E}-01$ |
| Thyroid | $4.3 \mathrm{E}-04 / 4.3 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $1.3 \mathrm{E}-05 / 1.3 \mathrm{E}-07$ |
| Bone Surface | $9.7 \mathrm{E}+02 / 9.7 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $2.9 \mathrm{E}+01 / 2.9 \mathrm{E}-01$ |
| Liver | $1.7 \mathrm{E}+02 / 1.7 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $1.0 \mathrm{E}+01 / 1.0 \mathrm{E}-01$ |
| Other | $1.6 \mathrm{E}+01 / 1.6 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $9.8 \mathrm{E}-01 / 9.8 \mathrm{E}-03$ |
| Lower Large Intestine | $3.5 \mathrm{E}-02 / 3.5 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $2.1 \mathrm{E}-03 / 2.1 \mathrm{E}-05$ |
| Upper Large Intestine | $1.2 \mathrm{E}-02 / 1.2 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $7.1 \mathrm{E}-04 / 7.1 \mathrm{E}-06$ |
| Small Intestine | $2.4 \mathrm{E}-03 / 2.4 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $1.5 \mathrm{E}-04 / 1.5 \mathrm{E}-06$ |
| Effective Dose Equivalent |  |  | $9.5 \mathrm{E}+01 / 9.5 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting and alpha spectrometry. The gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $310,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $95 \mathrm{rem} / 0.95 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $950,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $67 \mathrm{rem}(0.67 \mathrm{~Sv})$.

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $310,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 95 rem ( 0.95 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level ( 0.100 rem ) for members of the public. It is more than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). These dose levels are significant, although they were based on only one sample. However, follow-up urine sampling should be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$


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$\qquad$


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$\qquad$



Internal Dosimetry Evaluation Form

| NAME: ${ }^{(5)}$ (6) |  |  | SSN: ${ }^{(6)}$ (6) |  |
| :---: | :---: | :---: | :---: | :---: |
|  | Injection Absorption Not applicable | INTAKE DATE OR PERIOD: 1/29/66 through 2/14/66, onsite 2/6/66 |  |  |
| SUMMARY OF EXPOSURE CONDITIONS: <br> Radionuclides/Respiratory Class/Particle Size: ${ }^{239} \mathrm{Pu} / 100 \%$ Class Y/1 $\mu \mathrm{m}$ AMAD <br> Date or Period of Evaluated Data: 2 samples, 2/14/66 and 4/8/66 <br> Duration of Exposure: Unknown <br> Location of Exposure: Camp Wilson, near Palomares, Spain |  |  |  |  |
| EVALUATION DATA: <br> Air Sampling Health Physics Survey Data Bioassay - Urinalysis Fecal Nasal Smears In Vivo | $\square$ Attached $\square$ Attached $\boxtimes$ Attached $\square$ Attached $\boxtimes$ Attached $\square$ Attached | $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process | 《 Unavailable <br> 区 Unavailable Unavailable Unavailable Unavailable Unavailable |  |
| Medical Treatment: <br> Skin Decontamination: <br> Decorporation: <br> Catharsis: <br> Surgical excision: | $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ $\square \mathrm{Yes}$ | $\boxtimes$ No Date: <br> $\boxtimes$ No Agent: <br> $\boxtimes$ No Agent: <br> $\boxtimes$ No Date: |  | Date: <br> Date: $\qquad$ |

## EVALUATION METHODOLOGY:

Assumptions: Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class Y, $1 \mu \mathrm{~m}$ AMAD particle size on $2 / 6 / 66$
Code/Model used for: Intake Estimate: CINDY, Ver. 1.4/JONES
Dose Estimate: CINDY, Ver. 1.4/ICRP 30, Part 4, General Systemic Model
RESULTS SUMMARY
Estimated Intake Activity (pCi): 110000
50 YR CEDE (rem) : 34 ( 0.34 Sv )
Organ Dose Equivalent Summary 50 YR CDE (rem/Sv)

| Bone Surface | $340 / 3.4$ |
| :--- | ---: |
| Lung | $130 / 1.3$ |
| Liver | $61 / 0.61$ |
| Red Marrow | $26 / 0.26$ |
| Other | $5.8 / 0.058$ |
| Testes | $4.9 / 0.049$ |



| RECOMMENDATIONS: <br> Additional Bioassay Required <br> Suggested Sampling Frequency: <br> Work Restrictions:$\quad$ N/A |  | $\square$ Urinalysis | $\square$ Fecal | $\square$ ln Vivo |
| :--- | :--- | :--- | :--- | :--- |

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Internal Dosimetry Case Narrative

## Identification:



## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $2 / 6 / 66$. The date is the midpoint of the period on station from $1 / 29 / 66$ to $2 / 14 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1147$ | G | $2 / 14 / 66$ | NR | NR |  |
| $66-2872$ | AS | $4 / 8 / 66$ | 0.076 | 0.007 | $\checkmark$ |
| $66-2872$ | G | $4 / 8 / 66$ | 1.66 | 0.330 |  |
| * Geans gross alpha counting; AS means alpha spectrometry; | NR means no result reported. |  |  |  |  |

A nasal swipe (sample \#66-1333) was reported to have been taken for on $2 / 14 / 66$. No result was available for this sample.

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 110,000 | $34 / 0.34$ |
| LUDEP | 321,000 | $22 / 0.22$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> (rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $4.9 \mathrm{E}+00 / 4.9 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.2 \mathrm{E}+00 / 1.2 \mathrm{E}-02$ |
| Breast | $1.6 \mathrm{E}-04 / 1.6 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $2.4 \mathrm{E}-05 / 2.4 \mathrm{E}-07$ |
| Red Marrow | $2.6 \mathrm{E}+01 / 2.6 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $3.2 \mathrm{E}+00 / 3.2 \mathrm{E}-02$ |
| Lung | $1.3 \mathrm{E}+02 / 1.3 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.5 \mathrm{E}+01 / 1.5 \mathrm{E}-01$ |
| Thyroid | $1.5 \mathrm{E}-04 / 1.5 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $4.6 \mathrm{E}-06 / 4.6 \mathrm{E}-08$ |
| Bone Surface | $3.4 \mathrm{E}+02 / 3.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.0 \mathrm{E}+01 / 1.0 \mathrm{E}-01$ |
| Liver | $6.1 \mathrm{E}+01 / 6.1 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $3.7 \mathrm{E}+00 / 3.7 \mathrm{E}-02$ |
| Other | $5.8 \mathrm{E}+00 / 5.8 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $3.5 \mathrm{E}-01 / 3.5 \mathrm{E}-03$ |
| Lower Large Intestine | $1.2 \mathrm{E}-02 / 1.2 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $7.5 \mathrm{E}-04 / 7.5 \mathrm{E}-06$ |
| Upper Large Intestine | $4.2 \mathrm{E}-03 / 4.2 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $2.5 \mathrm{E}-04 / 2.5 \mathrm{E}-06$ |
| Small Intestine | $8.6 \mathrm{E}-04 / 8.6 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $5.1 \mathrm{E}-05 / 5.1 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $3.4 \mathrm{E}+01 / 3.4 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The sample analyzed by gross alpha counting only was not included in the analysis since no result was reported for it. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $110,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $34 \mathrm{rem} / 0.34 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $321,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of 22 rem ( 0.22 Sv ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $110,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 34 rem ( 0.34 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

## Prepared By:

Name:


Date: $\qquad$

Peer Reviewed By:
Name: $\qquad$
Signature:


Date: $\qquad$





Internal Dosimetry Evaluation Form


## RECOMIMENDATIONS:

Additional Bioassay Required
Suggested Sampling Frequency:
Work Restrictions:
$\square$ UrinalysisFecalIn Vivo

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## Internal Dosimetry Case Narrative

## Identification:



## Imcidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $2 / 13 / 66$. The date is the midpoint of the period on station from $2 / 1 / 66$ to $2 / 27 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-3411$ | AS | $2 / 27 / 66$ | 0.122 | 0.045 | $\checkmark$ |
| $66-3411$ | G | $2 / 27 / 66$ | NR | NR |  |
| $66-1494$ | AS | $2 / 28 / 66$ | 0.282 | 0.116 | $\checkmark$ |
| $66-1494$ | G | $2 / 28 / 66$ | 1.52 | 0.310 |  |
| * G means gross alpha counting; AS means alpha spectrometry; | NR means no result reported. |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CINDY | 190,000 | $58 / 0.58$ |
| LUDEP | 616,000 | $43 / 0.43$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> (rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $8.5 \mathrm{E}+00 / 8.5 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $2.1 \mathrm{E}+00 / 2.1 \mathrm{E}-02$ |
| Breast | $2.8 \mathrm{E}-04 / 2.8 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $4.2 \mathrm{E}-05 / 4.2 \mathrm{E}-07$ |
| Red Marrow | $4.6 \mathrm{E}+01 / 4.6 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $5.5 \mathrm{E}+00 / 5.5 \mathrm{E}-02$ |
| Lung | $2.2 \mathrm{E}+02 / 2.2 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $2.6 \mathrm{E}+01 / 2.6 \mathrm{E}-01$ |
| Thyroid | $2.7 \mathrm{E}-04 / 2.7 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $8.0 \mathrm{E}-06 / 8.0 \mathrm{E}-08$ |
| Bone Surface | $5.9 \mathrm{E}+02 / 5.9 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.8 \mathrm{E}+01 / 1.8 \mathrm{E}-01$ |
| Liver | $1.1 \mathrm{E}+02 / 1.1 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $6.4 \mathrm{E}+00 / 6.4 \mathrm{E}-02$ |
| Other | $1.0 \mathrm{E}+01 / 1.0 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $6.0 \mathrm{E}-01 / 6.0 \mathrm{E}-03$ |
| Lower Large Intestine | $2.2 \mathrm{E}-02 / 2.2 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $1.3 \mathrm{E}-03 / 1.3 \mathrm{E}-05$ |
| Upper Large Intestine | $7.2 \mathrm{E}-03 / 7.2 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $4.3 \mathrm{E}-04 / 4.3 \mathrm{E}-06$ |
| Small Intestine | $1.5 \mathrm{E}-03 / 1.5 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $8.9 \mathrm{E}-05 / 8.9 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $5.8 \mathrm{E}+01 / 5.8 \mathrm{E}-01$ |

Two urine samples were analyzed by gross alpha counting and alpha spectrometry. The gross alpha analyses was not included in the modeling since an alpha spectrometry result was available for the each sample. The results were fit using CINDY and the Jones excretion model, to estimate an intake (190,000 pCi ), organ doses, and a CEDE ( $58 \mathrm{rem} / 0.58 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $616,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of 43 rem ( 0.43 Sv ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $190,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 58 rem ( 0.58 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level ( 0.100 rem ) for members of the public. It is more than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not normally associated with these dose levels. However, follow-up urine sampling should be considered to provide additional assessment of the exposure.

## Prepared By:

Name:

Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$


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## (b) (6)

## Internal Dosimetry Evaluation Form



## RECOMMENDATIONS:

| Additional Bioassay Required |
| :--- |
| Suggested Sampling Frequency: |

Work Restrictions:
N/A

## Internal Dosimetry Case Narrative

## Identification:

## Name:

SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.

$$
\text { Radionuclide(s): }{ }^{239} \mathrm{Pu} \text {. }
$$

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 15 / 66$. The date is the midpoint of the period on station from 2/5/66 to 2/26/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1419$ | AS | $2 / 26 / 66$ | 0.163 | 0.024 | $\checkmark$ |
| $66-1419$ | G | $2 / 26 / 66$ | 0.731 | 0.216 |  |
| G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CLNDY | 140,000 | $43 / 0.43$ |
| LUDEP | 697,000 | $49 / 0.49$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $6.2 \mathrm{E}+00 / 6.2 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.6 \mathrm{E}+00 / 1.6 \mathrm{E}-02$ |
| Breast | $2.1 \mathrm{E}-04 / 2.1 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $3.1 \mathrm{E}-05 / 3.1 \mathrm{E}-07$ |
| Red Marrow | $3.4 \mathrm{E}+01 / 3.4 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $4.0 \mathrm{E}+00 / 4.0 \mathrm{E}-02$ |
| Lung | $1.6 \mathrm{E}+02 / 1.6 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.9 \mathrm{E}+01 / 1.9 \mathrm{E}-01$ |
| Thyroid | $2.0 \mathrm{E}-04 / 2.0 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $5.9 \mathrm{E}-06 / 5.9 \mathrm{E}-08$ |
| Bone Surface | $4.4 \mathrm{E}+02 / 4.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.3 \mathrm{E}+01 / 1.3 \mathrm{E}-01$ |
| Liver | $7.8 \mathrm{E}+01 / 7.8 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $4.7 \mathrm{E}+00 / 4.7 \mathrm{E}-02$ |
| Other | $7.4 \mathrm{E}+00 / 7.4 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $4.4 \mathrm{E}-01 / 4.4 \mathrm{E}-03$ |
| Lower Large Intestine | $1.6 \mathrm{E}-02 / 1.6 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $9.5 \mathrm{E}-04 / 9.5 \mathrm{E}-06$ |
| Upper Large Intestine | $5.3 \mathrm{E}-03 / 5.3 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $3.2 \mathrm{E}-04 / 3.2 \mathrm{E}-06$ |
| Small Intestine | $1.1 \mathrm{E}-03 / 1.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $6.5 \mathrm{E}-05 / 6.5 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $4.3 \mathrm{E}+01 / 4.3 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting and alpha spectrometry. The gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $140,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $43 \mathrm{rem} / 0.43 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $697,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $49 \mathrm{rem}(0.49 \mathrm{~Sv})$.

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $140,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $43 \mathrm{rem}(0.43$ Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level $(0.100 \mathrm{rem})$ for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

Prepared By:
Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

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Internal Dosimetry Evaluation Form


## RECOMMENDATIONS:

Additional Bioassay Required
Suggested Sampling Frequency:$\square$ UrinalysisFecal
$\square$ In Vivo
Work Restrictions:

## Internal Dosimetry Case Narrative

## Identification:

Name:
SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 18 / 66$. The date is the midpoint of the period on station from $1 / 27 / 66$ to $3 / 13 / 66$.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-2007$ | G | $3 / 14 / 66$ | 0.180 | 0.100 | $\checkmark$ |
| $66-3402$ | AS | $6 / 5 / 66$ | 0.049 | 0.037 | $\checkmark$ |
| $66-3402$ | G | $6 / 5 / 66$ | NR | NR |  |

* G means gross alpha counting; AS means alpha spectrometry; NR means no result reported.

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CINDY | 140,000 | $43 / 0.43$ |
| LUDEP | 265,000 | $19 / 0.19$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem $/$ Sv $)$ | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> $($ rem $/$ Sv |
| :--- | :---: | :---: | :---: |
| Testes | $6.2 \mathrm{E}+00 / 6.2 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.6 \mathrm{E}+00 / 1.6 \mathrm{E}-02$ |
| Breast | $2.1 \mathrm{E}-04 / 2.1 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $3.1 \mathrm{E}-05 / 3.1 \mathrm{E}-07$ |
| Red Marrow | $3.4 \mathrm{E}+01 / 3.4 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $4.0 \mathrm{E}+00 / 4.0 \mathrm{E}-02$ |
| Lung | $1.6 \mathrm{E}+02 / 1.6 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.9 \mathrm{E}+01 / 1.9 \mathrm{E}-01$ |
| Thyroid | $2.0 \mathrm{E}-04 / 2.0 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $5.9 \mathrm{E}-06 / 5.9 \mathrm{E}-08$ |
| Bone Surface | $4.4 \mathrm{E}+02 / 4.4 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.3 \mathrm{E}+01 / 1.3 \mathrm{E}-01$ |
| Liver | $7.8 \mathrm{E}+01 / 7.8 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $4.7 \mathrm{E}+00 / 4.7 \mathrm{E}-02$ |
| Other | $7.4 \mathrm{E}+00 / 7.4 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $4.4 \mathrm{E}-01 / 4.4 \mathrm{E}-03$ |
| Lower Large Intestine | $1.6 \mathrm{E}-02 / 1.6 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $9.5 \mathrm{E}-04 / 9.5 \mathrm{E}-06$ |
| Upper Large Intestine | $5.3 \mathrm{E}-03 / 5.3 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $3.2 \mathrm{E}-04 / 3.2 \mathrm{E}-06$ |
| Small Intestine | $1.1 \mathrm{E}-03 / 1.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $6.5 \mathrm{E}-05 / 6.5 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $4.3 \mathrm{E}+01 / 4.3 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. The results were fit using CINDY and the Jones excretion model, to estimate an intake ( $140,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $43 \mathrm{rem} / 0.43 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $265,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of 19 rem ( 0.19 Sv ).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $140,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 43 rem ( 0.43 Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level ( 0.100 rem ) for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$




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## Internal Dosimetry Evaluation Form

| NAME: ${ }^{(0)}$ (6) | SSN: ${ }^{(\mathrm{la})(6)}$ |  |  |  |
| :---: | :---: | :---: | :---: | :---: |
|  | $\begin{aligned} & \square \text { Injection } \\ & \square \text { Absorption } \\ & \square \text { Not applicable } \end{aligned}$ | INTAKE DATE OR PERIOD: 2/4/66 through 2/14/66, onsite 2/9/66 |  |  |
| SUMMARY OF EXPOSURE C <br> Radionuclides/Respiratory Class Date or Period of Evaluated Data Duration of Exposure: Unknown Location of Exposure: Camp Wi | ITIONS: icle Size: ${ }^{239} \mathrm{Pu} / 1$ samples, 2/17/66 an near Palomares, | \% Class Y/I 3/5/66 |  |  |
| EVALUATION DATA: <br> Air Sampling Health Physics Survey Data Bioassay - Urinalysis Fecal Nasal Smears In Vivo | $\square$ Attached $\square$ Attached $\boxed{\text { Attached }}$ $\square$ Attached $\boxed{\text { Attached }}$ $\square$ Attached | $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process $\square$ In Process | 区 Unavailable <br> U Unavailable Unavailable Unavailable Unavailable Unavailable |  |
| Medical Treatment: <br> Skin Decontamination: <br> Decorporation: <br> Catharsis: <br> Surgical excision: | $\square$ Yes $\square$ Yes $\square$ Yes $\square$ Yes | $\boxed{\text { No }}$ Date: <br> Q No Agent <br> X No Agent: <br> 区 No Date: |  | Date: <br> Date: $\qquad$ |

EVALUATION METHODOLOGY:
Assumptions: Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on 2/9/66
Code/Model used for: Intake Estimate: CINDY, Ver. 1.4/JONES
Dose Estimate: CINDY, Ver. 1.4/ICRP 30, Part 4, General Systemic Model
RESULTS SUMMARY
Estimated Intake Activity (pCi): 120000
50 YR CEDE ( rem ) : 55 ( 0.55 Sv )
Organ Dose Equivalent Summary 50 YR CDE (rem/Sv)
Bone Surface 370/3.7
Lung 140/1.4
Liver 67/0.67
Red Marrow 29/0.29
Other 6.3/0.063
$\begin{array}{ll}\text { Testes } & 5.4 / 0.054\end{array}$

| DOSE ASSESSOR: | DATE: | PEER REVIEWER: |
| :--- | :--- | :--- |
| Signature: |  | DATE: |
| Print Name: | Signature: |  |
| SSN: | Print Name: |  |

## RECOMIMIENDATIONS:

| Additional Bioassay Required |
| :--- |
| Suggested Sampling Frequency: $\quad \square$ Urinalysis $\quad \square$ Fecal $\quad \square$ In Vivo |

Work Restrictions: N/A

Release of this document is restricted under the provisions of the Privacy Act, 5 U.S.C. 552(a). C.2-356

Internal Dosimetry Case Narrative

## Identification:

Name:
SSN:

## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on 2/9/66. The date is the midpoint of the period on station from 2/4/66 to 2/14/66 .

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1122$ | G | $2 / 17 / 66$ | 0.190 | 0.160 | $\checkmark$ |
| $66-1554$ | AS | $3 / 5 / 66$ | 0.080 | 0.016 | $\checkmark$ |
| $66-1554$ | $G$ | $3 / 5 / 66$ | 1.50 | 0.330 |  |
| $*$ G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

A nasal swipe was also reported (sample \#66-1656) from $\square$ on $3 / 1 / 66$; however, no result was available.

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE (rem/Sv) |
| :--- | :---: | :---: |
| CINDY | 120,000 | $37 / 0.37$ |
| LUDEP | 348,000 | $24 / 0.24$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $(\mathrm{rem} / \mathrm{Sv})$ | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: | :---: |
| Testes | $5.4 \mathrm{E}+00 / 5.4 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $1.3 \mathrm{E}+00 / 1.3 \mathrm{E}-02$ |
| Breast | $1.8 \mathrm{E}-04 / 1.8 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $2.7 \mathrm{E}-05 / 2.7 \mathrm{E}-07$ |
| Red Marrow | $2.9 \mathrm{E}+01 / 2.9 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $3.5 \mathrm{E}+00 / 3.5 \mathrm{E}-02$ |
| Lung | $1.4 \mathrm{E}+02 / 1.4 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $1.6 \mathrm{E}+01 / 1.6 \mathrm{E}-01$ |
| Thyroid | $1.7 \mathrm{E}-04 / 1.7 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $5.0 \mathrm{E}-06 / 5.0 \mathrm{E}-08$ |
| Bone Surface | $3.7 \mathrm{E}+02 / 3.7 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.1 \mathrm{E}+01 / 1.1 \mathrm{E}-01$ |
| Liver | $6.7 \mathrm{E}+01 / 6.7 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $4.0 \mathrm{E}+00 / 4.0 \mathrm{E}-02$ |
| Other | $6.3 \mathrm{E}+00 / 6.3 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $3.8 \mathrm{E}-01 / 3.8 \mathrm{E}-03$ |
| Lower Large Intestine | $1.4 \mathrm{E}-02 / 1.4 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $8.2 \mathrm{E}-04 / 8.2 \mathrm{E}-06$ |
| Upper Large Intestine | $4.6 \mathrm{E}-03 / 4.6 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $2.7 \mathrm{E}-04 / 2.7 \mathrm{E}-06$ |
| Small Intestine | $9.4 \mathrm{E}-04 / 9.4 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $5.6 \mathrm{E}-05 / 5.6 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $3.7 \mathrm{E}+01 / 3.7 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting, and the other was analyzed by both gross alpha counting and alpha spectrometry. The gross alpha analysis for the second sample was not included in the modeling since an alpha spectrometry result was available for the same sample. The results were fit using CINDY and the Jones excretion model, to estimate an intake ( $120,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( 37 rem/0.37 Sv; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $348,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of 24 rem ( 0.24 Sv ).

[^5]
## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $120,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $37 \mathrm{rem}(0.37$ Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is less than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not associated with that dose level. However, follow-up urine sampling now could be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$



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## Internal Dosimetry Evaluation Form

| NAME: (b) (6) | SSN: NA |  |  |  |
| :---: | :---: | :---: | :---: | :---: |
|  | Injection Absorption Not applicable | INTAKE DATE OR PERIOD: 2/5/66 through 2/26/66, onsite 2/15/66 |  |  |
| SUMMMARY OF EXPOSURE C <br> Radionuclides/Respiratory Clas <br> Date or Period of Evaluated Dat <br> Duration of Exposure: Unknown <br> Location of Exposure: Camp W | ITIONS: icle Size: ${ }^{239} \mathrm{Pu} / 1$ ample, 2/26/66 near Palomares, | \% Class Y/1 $\mu$ <br> ain |  |  |
| EVALUATION DATA: <br> Air Sampling Health Physics Survey Data Bioassay - Urinalysis Fecal Nasal Smears In Vivo | $\square$ Attached $\square$ Attached Q Attached $\square$ Attached $\square$ Attached $\square$ Attached | In Process In Process In Process In Process In Process In Process | X Unavailable <br> X Unavailable Unavailable Unavailable Unavailable Unavailable |  |
| Medical Treatment: <br> Skin Decontamination: <br> Decorporation: <br> Catharsis: <br> Surgical excision: | $\square$ Yes $\square$ Yes $\square$ Yes $\square$ Yes | $\boxtimes$ No Date: <br> $\boxed{N}$ No Agent: <br> $\boxed{\text { No }}$ Agent: <br> $\boxtimes$ No Date: |  | Date: <br> Date: |

## EVALUATION METHODOLOGY:

Assumptions: Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 15 / 66$
$\begin{array}{ll}\text { Code/Model used for: } & \begin{array}{l}\text { Intake Estimate: CINDY, Ver. 1.4/JONES } \\ \\ \\ \\ \text { Dose Estimate: CINDY, Ver. 1.4/ICRP 30, Part 4, General Systemic Model }\end{array}\end{array}$
RESULTS SUMMARY

| Estimated Intake Activity (pCi): 185000 |  |  |
| :--- | :--- | :--- |
| 50 YR CEDE (rem) : $55(0.55$ Sv) | 50 YR CDE (rem/Sv) |  |
| Organ Dose Equivalent Summary | $560 / 5.6$ |  |
| Bone Surface | $210 / 2.1$ |  |
| Lung | $100 / 1$ |  |
| Liver | $43 / 0.43$ |  |
| Red Marrow | $9.5 / 0.095$ |  |
| Other | $8 / 0.08$ |  |
| Testes | PEER REVIEWER: | DATE: |
| DOSE ASSESSOR: |  |  |

Signature: $\qquad$ Signature: $\qquad$
Print Name: $\qquad$
SSN:
Print Name: $\qquad$

## RECOMMIENDATIONS:

Additional Bioassay Required
Suggested Sampling Frequency:

Work Restrictions: N/A

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# Internal Dosimetry Case Narrative 

## Identification:



## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of $\mathrm{Pu}-239 ; 100 \%$ Class $\mathrm{Y} ; 1 \mu \mathrm{~m}$ AMAD particle size on $2 / 15 / 66$. The date is the midpoint of the period on station from 2/5/66 to 2/26/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> $(\mathrm{pCi} /$ day $)$ | Error <br> $(\mathrm{pCi} /$ day $)$ | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-1438$ | AS | $2 / 26 / 66$ | 0.208 | 0.086 | $\checkmark$ |
| $66-1438$ | G | $2 / 26 / 66$ | 1.51 | 0.310 |  |
| G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

[^6]Revised Dose Evaluation Report

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake $(\mathrm{pCi})$ | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 185,000 | $55 / 0.55$ |
| LUDEP | 888,000 | $62 / 0.62$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $($ rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $8.0 \mathrm{E}+00 / 8.0 \mathrm{E}-02$ | $2.5 \mathrm{E}-01$ | $2.0 \mathrm{E}+00 / 2.0 \mathrm{E}-02$ |
| Breast | $2.7 \mathrm{E}-04 / 2.7 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $4.0 \mathrm{E}-05 / 4.0 \mathrm{E}-07$ |
| Red Marrow | $4.3 \mathrm{E}+01 / 4.3 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $5.2 \mathrm{E}+00 / 5.2 \mathrm{E}-02$ |
| Lung | $2.1 \mathrm{E}+02 / 2.1 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $2.5 \mathrm{E}+01 / 2.5 \mathrm{E}-01$ |
| Thyroid | $2.5 \mathrm{E}-04 / 2.5 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $7.5 \mathrm{E}-06 / 7.5 \mathrm{E}-08$ |
| Bone Surface | $5.6 \mathrm{E}+02 / 5.6 \mathrm{E}+00$ | $3.0 \mathrm{E}-02$ | $1.7 \mathrm{E}+01 / 1.7 \mathrm{E}-01$ |
| Liver | $1.0 \mathrm{E}+02 / 1.0 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $6.0 \mathrm{E}+00 / 6.0 \mathrm{E}-02$ |
| Other | $9.5 \mathrm{E}+00 / 9.5 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $5.7 \mathrm{E}-01 / 5.7 \mathrm{E}-03$ |
| Lower Large Intestine | $2.0 \mathrm{E}-02 / 2.0 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $1.2 \mathrm{E}-03 / 1.2 \mathrm{E}-05$ |
| Upper Large Intestine | $6.9 \mathrm{E}-03 / 6.9 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $4.1 \mathrm{E}-04 / 4.1 \mathrm{E}-06$ |
| Small Intestine | $1.4 \mathrm{E}-03 / 1.4 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $8.4 \mathrm{E}-05 / 8.4 \mathrm{E}-07$ |
| Effective Dose Equivalent |  |  | $5.5 \mathrm{E}+01 / 5.5 \mathrm{E}-01$ |

One urine sample was analyzed by gross alpha counting and alpha spectrometry. The gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $185,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $55 \mathrm{rem} / 0.55 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $888,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of $62 \mathrm{rem}(0.62 \mathrm{~Sv})$.

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $185,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $55 \mathrm{rem}(0.55$ Sv ). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is $10 \%$ more than the working lifetime limit of 50
rem recommended by the National Council on Radiation Protection and Measurements (NCRP). Serious health effects are not normally associated with these dose levels. However, follow-up urine sampling should be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$


Revised Dose Evaluation Report April 2001

Internal Dosimetry Evaluation Form


| EVALUATION DATA： |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: |
| Air Sampling | $\square$ Attached | $\square$ In Process |  | 区 Unavailable |  |
| Health Physics Survey Data | $\square$ Attached | $\square$ In Process |  |  |  |
| Bioassay－Urinalysis | $\triangle$ Attached | $\square$ In Process |  | $\square$ Unavailable |  |
| Fecal | $\square$ Attached | $\square$ In Process |  | 区 Unavailable |  |
| Nasal Smears | $\square$ Attached | $\square$ In Process |  | Unavailable |  |
| In Vivo | $\square$ Attached | $\square$ In Process |  | 区 Unavailable |  |
| Medical Treatment： |  |  |  |  |  |
| Skin Decontamination： | $\square \mathrm{Yes}$ | 区No | Date： |  |  |
| Decorporation： | $\square$ Yes | No | Agent： |  | Date： |
| Catharsis： | $\square \mathrm{Yes}$ | No | Agent： |  | Date： |
| Surgical excision： | $\square \mathrm{Yes}$ | ® No | Date： |  |  |


| EVALUATION METHODOLOGY： |  |
| :--- | :--- |
| Assumptions： | Acute inhalation intake of ${ }^{239} \mathrm{Pu}, 100 \%$ Class $\mathrm{Y}, 1 \mu \mathrm{~m}$ AMAD particle size on 2／17／66 |
| Code／Model used for： | Intake Estimate： <br>  <br>  <br> Dose Estimate： |


| RESULTS SUMMARY |  |
| :--- | :---: |
| Estimated Intake Activity（pCi）： | 4,400 |
| 50 YR CEDE（rem）： $1.4(0.014 \mathrm{~Sv}$ ） |  |
| Organ Dose Equivalent Summary | $\mathbf{5 0}$ YR CDE（rem／Sv） |
| Bone Surface | $14 / 0.14$ |
| Lung | $5 / 0.05$ |
| Liver | $2.5 / 0.025$ |
| Red Marrow | $1.1 / 0.011$ |
| Other | $0.2 / 0.002$ |
| Testes | $0.2 / 0.002$ |


| DOSE ASSESSOR： | DATE： | PEER REVIEWER： | DATE： |
| :---: | :---: | :---: | :---: |
| Signature： |  | Signature： |  |
| Print Name： |  | Print Name： |  |
| SSN： |  | SSN： |  |


| RECOMMENDATIONS： |  |
| :--- | :--- | :--- | :--- | :--- |
| Additional Bioassay Required <br> Suggested Sampling Frequency： <br> Work Restrictions： | N／A |

## Internal Dosimetry Case Narrative

## Identification:

> Name:

SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.

## Assumptions/Basis/Data Sources:

Acute inhalation intake of Pu-239; $100 \%$ Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $2 / 17 / 66$. The date is the midpoint of the period on station from 2/6/66 to 2/28/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-2446$ | G | $3 / 8 / 66$ | 1.49 | 0.790 |  |
| $66-3273$ | AS | $6 / 1 / 66$ | ND | ND | $\checkmark$ |
| $66-3273$ | G | $6 / 1 / 66$ | NR | NR |  |
| G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 4,400 | $1.4 / 0.014$ |
| LUDEP | 12,400 | $0.9 / 0.009$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> $(\mathrm{rem} /$ Sv $)$ | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/Sv) |
| :--- | :---: | :---: | :---: |
| Testes | $2.0 \mathrm{E}-01 / 2.0 \mathrm{E}-03$ | $2.5 \mathrm{E}-01$ | $4.9 \mathrm{E}-02 / 4.9 \mathrm{E}-04$ |
| Breast | $6.5 \mathrm{E}-06 / 6.5 \mathrm{E}-08$ | $1.5 \mathrm{E}-01$ | $9.8 \mathrm{E}-07 / 9.8 \mathrm{E}-09$ |
| Red Marrow | $1.1 \mathrm{E}+00 / 1.1 \mathrm{E}-02$ | $1.2 \mathrm{E}-01$ | $1.3 \mathrm{E}-01 / 1.3 \mathrm{E}-03$ |
| Lung | $5.0 \mathrm{E}+00 / 5.0 \mathrm{E}-02$ | $1.2 \mathrm{E}-01$ | $6.0 \mathrm{E}-01 / 6.0 \mathrm{E}-03$ |
| Thyroid | $6.1 \mathrm{E}-06 / 6.1 \mathrm{E}-08$ | $3.0 \mathrm{E}-02$ | $1.8 \mathrm{E}-07 / 1.8 \mathrm{E}-09$ |
| Bone Surface | $1.4 \mathrm{E}+01 / 1.4 \mathrm{E}-01$ | $3.0 \mathrm{E}-02$ | $4.1 \mathrm{E}-01 / 4.1 \mathrm{E}-03$ |
| Liver | $2.5 \mathrm{E}+00 / 2.5 \mathrm{E}-02$ | $6.0 \mathrm{E}-02$ | $1.5 \mathrm{E}-01 / 1.5 \mathrm{E}-03$ |
| Other | $2.3 \mathrm{E}-01 / 2.3 \mathrm{E}-03$ | $6.0 \mathrm{E}-02$ | $1.4 \mathrm{E}-02 / 1.4 \mathrm{E}-04$ |
| Lower Large Intestine | $5.0 \mathrm{E}-04 / 5.0 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $3.0 \mathrm{E}-05 / 3.0 \mathrm{E}-07$ |
| Upper Large Intestine | $1.7 \mathrm{E}-04 / 1.7 \mathrm{E}-06$ | $6.0 \mathrm{E}-02$ | $1.0 \mathrm{E}-035 / 1.0 \mathrm{E}-037$ |
| Small Intestine | $3.4 \mathrm{E}-05 / 3.4 \mathrm{E}-07$ | $6.0 \mathrm{E}-02$ | $2.1 \mathrm{E}-06 / 2.1 \mathrm{E}-08$ |
| Effective Dose Equivalent |  |  | $1.4 \mathrm{E}+00 / 1.4 \mathrm{E}-02$ |

One urine sample was analyzed by gross alpha counting only, and the other was analyzed by both gross alpha counting and alpha spectrometry. For the sample analyzed with both methods, the gross alpha analysis was not included in the modeling since no result was reported and an alpha spectrometry result was available for the same sample. The alpha spectrometry sample result was reported as No Detectable Activity. A value of 0.003 pCi was used to represent this outcome. The gross alpha sample was also excluded because it was suspected of contamination during sample collection on the site and because it did not fit the expected pattern of urinary excretion. The result was fit using CINDY and the Jones excretion model, to estimate an intake ( $4,400 \mathrm{pCi}$ ), organ doses, and a CEDE ( $1.4 \mathrm{rem} / 0.14 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $12,400 \mathrm{pCi}$ and a CEDE (ICRP-60) of $0.9 \mathrm{rem}(0.009 \mathrm{~Sv})$.

In a separate run that used the gross alpha result, CINDY produced estimated intake and CEDE of $890,000 \mathrm{pCi}$ and $270 \mathrm{rem}(2.7 \mathrm{~Sv})$ respectively. However, these estimates were not considered realistic for the reasons stated above.

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $4,400 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of $1.4 \mathrm{rem}(0.014$ Sv). That dose is much less than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the
current level ( 0.100 rem ) for members of the public. Serious health effer current level ( 0.100 rem ) for members of the public. Serious health effects are not normally associated with these dose levels.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

Peer Reviewed By:
Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$

[^7]

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|  |  |  |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
|  |  |  |  |  |  |  |  |  |
| PVT |  |  |  |  |  |  |  |  |
| $66-2146$ |  |  |  |  |  |  |  |  |
| 8ASE (87-60) Giessen |  | $\begin{aligned} & \hline \text { OCCUPATION (61-62) } \\ & 57 E 20 \\ & \hline \end{aligned}$ |  |  <br>  |  |  |  |  |
| DATE RECEIVED 7 April 1966 |  | SAMPLE VOLUME 750 |  |  |  |  |  |  |
| TECHNIGIAN (sig mature (ib) (6) |  |  |  H M Hux |  |  |  |  |  |
| URINE |  | $\cdots$ |  |  |  |  |  |  |
| Counter Number | 38*7 |  |  |  |  | Countor Number |  | - |
| Counter Bkg. (epm) | 0.03 |  |  |  |  | "Countar Bkg. |  |  |
| Countor Eff. (\%) | 0.45 |  |  |  | - | Countor Eff. |  |  |
| Date/TIme - Start | $4-28-66$ |  |  |  |  | Daie/TIme - Stort |  |  |
| -Stop |  |  | , |  |  |  |  |  |
| Total Counts | 1411 |  |  |  |  | Toial Counts |  |  |
| Counting Tlme | 960 |  |  |  |  | Counting Tima |  |  |
| Gross cpm | 1.47 |  | h. |  |  | CGrós ${ }^{\text {cospm }}$ |  |  |
| Bkg. Cpm | 0.07 |  |  |  |  | 8kkg cpm |  | 92PC |
| Net cpm | 1.44 |  | $\cdots$ | Not mv/ see ${ }^{\text {a }}$ |  | Hoot epm |  | 28BB |
| dpm. | 3,20 |  | 4 | curlos m mve | 3 |  |  | C- |
| dpm/24 hr. (69.74), |  |  | , , $7^{2}$, |  |  | $\mathrm{dpg} / \mathrm{cc}$ co- |  |  |
| K-40 Corrretiorpull | $1.92 \pm 0.11$ |  | $\therefore$ | that fers |  | Noutron Doso (rads) (63. |  |  |
| Not-arderiggel | $1.44 \pm 0.1$ |  | \% |  |  | We7'ma $(69-74)$ |  |  |
| $\mathrm{D}(\mathrm{q})(63.68)$ |  |  |  |  | 焱 | $D(\mathrm{~g})(63.68)$ |  |  |




Internal Dosimetry Evaluation Form


## RECOMMENDATIONS:

| Additional Bioassay Required |
| :--- |
| Suggested Sampling Frequency: |
| Work Restrictions: |$\quad$ N/A $\quad \square$ Urinalysis $\quad \square$ Fecal $\quad \square$ In Vivo

[^8]Internal Dosimetry Case Narrative

## Identification:

Name:
SSN:


## Incidents:

Individual participated on site in response duties resulting from an accident involving three nuclear weapons at Palomares, Spain on January 17, 1966. Individual may have been exposed to weapons materials (primarily plutonium-239) by inhalation and ingestion from contaminated weapon and aircraft debris, lands, and vegetation. Primary activities included search, radiological monitoring, recovery of accident debris, and processing for disposal.

## Previous Intake/Dose Assessments:

This assessment applies to Palomares accident activities only. No previous intakes or doses were considered.

## Other Information:

None.
Radionuclide(s): ${ }^{239} \mathrm{Pu}$.
Assumptions/Basis/Data Sources:
Acute inhalation intake of Pu-239; 100\% Class Y; $1 \mu \mathrm{~m}$ AMAD particle size on $3 / 26 / 66$. The date is the midpoint of the period on station from 3/14/66 to 4/8/66.

Inhalation was assumed as the major route of entry because the primary contaminant was created by explosion and fire and deposited in sandy soil and on buildings and plants. Conditions were generally windy and significant activity was underway.

Dose was determined entirely from modeling intake based on the following urinalysis results for this individual. This individual's sample was identified for a follow-up analysis using alpha spectrometry after the initial gross alpha result was reviewed. That is, the initial urine sample for this individual was reprocessed radiochemically for alpha spectrometry. Results of the follow-up alpha spectrometry analysis are reported below and were used in preparing the dose estimate.

| Sample | Analysis* | Sample <br> Date | Result <br> (pCi/day) | Error <br> (pCi/day) | Included |
| :--- | :---: | :---: | :---: | :---: | :---: |
| $66-2860$ | AS | $4 / 8 / 66$ | 0.296 | 0.016 | $\checkmark$ |
| $66-2860$ | G | $4 / 8 / 66$ | 0.619 | 0.203 |  |
| $66-3241$ | AS | $5 / 26 / 66$ | NR | NR |  |
| $66-3241$ | G | $5 / 2666$ | 0.423 | 0.423 | $\checkmark$ |
| * G means gross alpha counting; AS means alpha spectrometry. |  |  |  |  |  |

Intakes and estimates of dose were prepared using CINDY Version 1.4 and LUDEP Version 2.05.
Intake was estimated using the Jones excretion model in CINDY and LUDEP.
Inhalation intake was estimated using the ICRP 66 respiratory tract model in LUDEP.
CINDY estimated dose derived using the ICRP 30, Part 4, General Systemic Model and weighting factors; and LUDEP used the recommendations of ICRP 60.

## Modeling:

CINDY and LUDEP were used to estimate the intake and dose with the following results:

| Model | Intake (pCi) | CEDE $(\mathrm{rem} / \mathrm{Sv})$ |
| :--- | :---: | :---: |
| CINDY | 400,000 | $120 / 1.2$ |
| LUDEP | $1,280,000$ | $90 / 0.9$ |

Doses to individual organs and estimation of the effective dose equivalent using CINDY reported the following results:

| Organ | Dose Equivalent <br> (rem/Sv) | Weighting <br> Factors | Weighted Organ <br> Dose Equivalent <br> (rem/sv) |
| :--- | :---: | :---: | :---: |
| Testes | $1.8 \mathrm{E}+01 / 1.8 \mathrm{E}-01$ | $2.5 \mathrm{E}-01$ | $4.5 \mathrm{E}+00 / 4.5 \mathrm{E}-02$ |
| Breast | $5.9 \mathrm{E}-04 / 5.9 \mathrm{E}-06$ | $1.5 \mathrm{E}-01$ | $8.9 \mathrm{E}-05 / 8.9 \mathrm{E}-07$ |
| Red Marrow | $9.6 \mathrm{E}+01 / 9.6 \mathrm{E}-01$ | $1.2 \mathrm{E}-01$ | $1.2 \mathrm{E}+01 / 1.2 \mathrm{E}-01$ |
| Lung | $4.6 \mathrm{E}+02 / 4.6 \mathrm{E}+00$ | $1.2 \mathrm{E}-01$ | $5.5 \mathrm{E}+01 / 5.5 \mathrm{E}-01$ |
| Thyroid | $5.6 \mathrm{E}-04 / 5.6 \mathrm{E}-06$ | $3.0 \mathrm{E}-02$ | $1.7 \mathrm{E}-05 / 1.7 \mathrm{E}-07$ |
| Bone Surface | $1.2 \mathrm{E}+03 / 1.2 \mathrm{E}+01$ | $3.0 \mathrm{E}-02$ | $3.7 \mathrm{E}+01 / 3.7 \mathrm{E}-01$ |
| Liver | $2.2 \mathrm{E}+02 / 2.2 \mathrm{E}+00$ | $6.0 \mathrm{E}-02$ | $1.3 \mathrm{E}+01 / 1.3 \mathrm{E}-01$ |
| Other | $2 . \mathrm{E}+01 / 2.1 \mathrm{E}-01$ | $6.0 \mathrm{E}-02$ | $1.3 \mathrm{E}+00 / 1.3 \mathrm{E}-02$ |
| Lower Large Intestine | $4.5 \mathrm{E}-02 / 4.5 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $2.7 \mathrm{E}-03 / 2.7 \mathrm{E}-05$ |
| Upper Large Intestine | $1.5 \mathrm{E}-02 / 1.5 \mathrm{E}-04$ | $6.0 \mathrm{E}-02$ | $9.1 \mathrm{E}-04 / 9.1 \mathrm{E}-06$ |
| Small Intestine | $3.1 \mathrm{E}-03 / 3.1 \mathrm{E}-05$ | $6.0 \mathrm{E}-02$ | $1.9 \mathrm{E}-04 / 1.9 \mathrm{E}-06$ |
| Effective Dose Equivalent |  |  | $1.2 \mathrm{E}+02 / 1.2 \mathrm{E}+00$ |

Two urine samples were analyzed by gross alpha counting and alpha spectrometry. For the first sample, the gross alpha analysis was not included in the modeling since an alpha spectrometry result was available for the same sample. For the second sample, no result was reported for the alpha spectrometry analysis, so the gross alpha counting result was used. The results were fit using CINDY and the Jones excretion model, to estimate an intake ( $400,000 \mathrm{pCi}$ ), organ doses, and a CEDE ( $120 \mathrm{rem} / 1.2 \mathrm{~Sv}$; ICRP-30) as shown above. LUDEP was also used to estimate an intake of $1,280,000 \mathrm{pCi}$ and a CEDE (ICRP-60) of
90 rem ( 0.9 Sv ). 90 rem (0.9 Sv).

## Conclusion:

Based on the results of intake estimates and dose calculations, this individual received an estimated intake of about $400,000 \mathrm{pCi}$ of ${ }^{239} \mathrm{Pu}$ resulting in a 50 -year committed effective dose equivalent of 120 rem ( 1.2

Sv). That dose is more than the cumulative dose ( 7 rem ) from a lifetime ( 70 years) of exposure at the current level ( 0.100 rem ) for members of the public. It is more than the working lifetime limit of 50 rem recommended by the National Council on Radiation Protection and Measurements (NCRP). These estimated dose levels are significant, although they were based on one sample that may have been collected on-site. However, follow-up urine sampling should be considered to provide additional assessment of the exposure.

## Prepared By:

Name: $\qquad$
Signature: $\qquad$

## Peer Reviewed By:

Name: $\qquad$
Signature: $\qquad$ Date: $\qquad$







## Appendix C. 3

## Contamination Cutoff Cases

Portions of this report have been designated records subject to the restriction of the Privacy Act, 5 U.S.C. 552(a) and are so marked.

## APPENDIX C. 3 Contamination Cutoff Cases

This section contains the intake and dose estimates for individuals whose urinalysis results were categorized as below a "contamination cutoff" of 0.1 picocuries per day ( $\mathrm{pCi} / \mathrm{d}$ ). These individuals primarily submitted samples while on-site at Palomares. Most of these initial samples were analyzed by the gross alpha procedure. The results of the analysis were less than $0.1 \mathrm{pCi} / \mathrm{d}$ and were evaluated for intake and dose. Most samples collected became contaminated with plutonium because of limited controls on spread of the very low amounts required to indicate a positive urinalysis result. The main body of the report contains a discussion on the problem of sample contamination and the "contamination cutoff".

This "contamination cutoff" group consisted of 314 individuals. Their urinalysis results ranged from 0.002 to $0.099 \mathrm{pCi} / \mathrm{d}$ for those processed by the gross alpha procedure and from 0.018 to $0.097 \mathrm{pCi} /$ day for those processed by alpha spectrometry. Intakes ranged from 1,500 to 150,000 picocuries and produced 50-year committed effective dose equivalents (CEDEs) of 0.46 to 46 rem ( 0.0046 to 0.46 Sv ). Table C.3-1 shows the distribution of CEDE for this group and indicates that most individuals' doses were relatively low. This section contains a listing of the results of the assessments. Individual narrative summaries were not prepared for these

Table C.3-1 Distribution of effective doses.

| CEDE Range <br> (rem) | Number of Cases |
| :---: | :---: |
| $0-10$ | 149 |
| $10-20$ | 94 |
| $20-30$ | 55 |
| $30-40$ | 14 |
| $40-50$ | 2 |

individuals.

The listing requires some explanatory notes to clarify features of the data presented. These notes include the following.

- An entry of " $\mathrm{n} / \mathrm{a}$ " means that data were not available in any of the records reviewed generally because no entry was recorded.
- An entry of " NR " means that a result for an analysis was not recorded on the appropriate data form.
- An entry of "ND" means that an analytical result was recorded as No Detectable Activity (NDA).
- An entry of "(12-hr)" in a Sample Volume cell means that a data form for the sample noted that the sample was collected for 12 hours.
- A shaded cell represents a result for a sample (collected on site) that exceeded $0.1 \mathrm{pCi} /$ day; the established maximum for modeling individuals in the "Contamination Cutoff" category.
- An entry of "mean" represents the average value for intake or CEDE from two or more sample results for the same person.


|  | NAME | Results of Modeling Individuals with Samples Assumed to be＂Uncontaminated．＂ |  |  |  |  |  |  |  |  | Coll shaded＝Addlionel sampla for indiv not modolod sinca $>0.1$ pCl／sampla， NR ，or ND |  |  | 50．YEAR CEDE （rom／Sv） |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
|  |  |  |  |  |  |  |  |  |  |  |  |  |  |  |
|  |  | SSN | $\begin{gathered} \text { START } \\ \text { EXPOSURE } \\ \text { DATE } \\ \hline \end{gathered}$ | $\begin{gathered} \text { END EXPOSURE } \\ \text { DATE } \\ \hline \end{gathered}$ | $\begin{gathered} \text { ESTMMATED } \\ \text { ACUTE } \\ \text { EXPOSURE } \\ \text { DATE } \\ \hline \end{gathered}$ | SAMPLE甘 | $\begin{gathered} \text { SAMPLEE } \\ \text { DATE } \\ \hline \end{gathered}$ | $\begin{gathered} \text { ELAPSED } \\ \text { DAYS } \\ \hline \end{gathered}$ | SAMPLE VOLUME （mL） | ANALYSIS | $\begin{gathered} \text { RESULT } \\ \text { (pCll } \\ \text { samplo) } \\ \hline \end{gathered}$ | ＋1／（pCll sampla） | $\begin{gathered} \text { INTAKE (1000s } \\ \mathrm{pCl}) \end{gathered}$ |  |
| － | （b）（6） | （b）（6） | 02／11／68 | 03／19／66 | 03／01／38 | 60－2200 | 03／19／66 | 18 | 900 | Gross Alpha | 0.035 | $8750 \mathrm{E}-03$ | 350 | $11 / 0.11$ |
| $\bigcirc$ |  |  | 0205／66 | 02／25／66 | 02115／68 | 66－1340 | 02／25／66 | 10 | 1300 | Gross Alpha | 0.022 | $1.850 \mathrm{E}-02$ | 18.0 | 5.510 .055 |
| ¢ |  |  | 02121／66 | 03／18／66 | 03／05／66 | 66－2258 | 03／18／66 | 13 | 780 | Gross Apha | 0.034 | 7.260 E .02 | 31.0 | $95 / 0095$ |
| （1） |  |  | 02／21166 | 03／19／68 | 03／06／68 | 66－2329 | 03／19／66 | 13 | 700 | Gross Aphia | 0.021 | $7.350 \mathrm{E}-02$ | 19.0 | 580.058 |
| $\cdots$ |  |  | 02113／66 | 03／00166 | $02 / 24168$ | 86－2425 | 03／08／66 | 12 | 1000 | Gross Apha | 0.010 | $2.740 \mathrm{E}-02$ | 92 | 2810.028 |
|  |  |  | 02121／66 | 03／11／66 | 03／05／6\％ | 66－2245 | 03／18／66 | 13 | 1000 | Gross Alpha | 0.098 | $1.090 \mathrm{E} \cdot 01$ | 890 | $27 / 0.27$ |
| 9 |  |  | 02108／66 | 02／28136 | 02111／68 | 66－1357 | 02128／66 | 10 | 950 （12．hr） | Gross Alpha | 0.032 | 7．875E－03 | 520 | 16／0．18 |
| 근． |  |  | 03／14／66 | 03／19／66 | 03／18／66 | 66－2333 | 03／19／66 | 3 | 900 | Gross Alpha | 0.096 | $1.017 \mathrm{E}-01$ | 200 | 6 610．061 |
| $\stackrel{\rightharpoonup}{0}$ |  |  |  |  |  | 66－2357 | 03／04／68 |  | 900 | Gross Apha | 0.15 | 1．190E－01 |  |  |
| ᄃ | （b）（6） | （b）（6） | 01／18166 | 02103／83 | 01／26166 | 66－1823 | 02／03／68 | 8 | 800 | Gross Apha | 0.018 | $4500 \mathrm{E}-03$ | 130 | 4／0．04 |
| \％ |  |  | 02／11／68 | 03／08／66 | 02／2366 | 66－2454 | 03／08／63 | 13 | 1000 | Gross Alpha | 0.085 | $2.460 \mathrm{E}-02$ | 77.0 | 24／0．24 |
| ¢ |  |  |  |  |  | 66－3121 | 04／13／68 |  | 700 | Gross Apha | 0.364 | 1.340 E .01 |  |  |
| 守 | （b）（6） | （b）（6） | 02109／66 | 03108／66 | 02／22166 | 66－1889 | 03／08／68 | 14 | 940 | Gross Apha | 0.031 | 1.960 E .02 | 290 | 8．910．089 |
| （1） |  |  | 01／11／68 | 03108／68 | $02111 / 6$ | 66－2615 | 03／08／68 | 25 | 850 | Gross Alpha | 0.070 | 7．900E．02 | 77.0 | $24 / 0.24$ |
| $\bigcirc$ |  |  | 02／13／66 | 03108／66 | 02124／66 | 66－2471 | 03／08／66 | 12 | 1000 | Gross Alpha | 0.077 | 1．913E．02 | 680 | 21／0．21 |
| ¢ |  |  | 02／09／68 | 03／08／68 | 02122／66 | 66－1882 | 03／08／66 | 14 | 750 | Gross Alpha | 0.017 | 2.000 E .02 | 160 | 4.900 .049 |
| － |  | （b）（6） | 02／10／66 | 03／19168 | 02128166 | 86－2208 | 03／19／66 | 19 | 900 | Gross Apha | 0.093 | 1．017E． 01 | 940 | 2910.29 |
| ¢ |  |  | 01／17／66 | 01／27166 | 01／22／66 | 66－1234 | 02119／66 | 28 | 1000 | Gross Apha | 0.036 | $2.120 \mathrm{E}-02$ | 41.0 | 13／0．13 |
| $\bigcirc$ |  |  | 01／18／66 | 02103／66 | 01／28166 | 66－1835 | 0203／66 | 8 | 750 | Gross Apha | 0.018 | $4.500 \mathrm{E}-03$ | 130 | 410.04 |
| 菅 |  |  | 02／24／66 | 0319／66 | 03／07766 | 66－2552 | 03／19／66 | 12 | 820 | Gross Alpha | 0.049 | 7.140 E .02 | 430 | 13／0． 13 |
|  |  |  | 02／21／68 | 03／12966 | 03／05／68 | 66－2088 | 03／18／68 | 13 | 860 | Gross Aloha | 0.098 | 7.640 E .02 | 87.0 | 27.027 |
| $\stackrel{7}{7}$ |  |  | 01／18／66 | 01／20／66 | 01／19／68 | 66－1132 | 01／20／66 | 1 | rua | Gross Alpha | 0.029 | 1.930 E .02 | 25 | 077100077 |
| 00 |  |  | 01／11／66 | 03／19／66 | 0217768 | 66－2337 | 03／19／68 | 30 | 800 | Gross Apha | 0.016 | $9.120 \mathrm{E}-02$ | 19.0 | $5 \% 0.058$ |
| $\bigcirc$ |  |  | 0211066 | 03／29／66 | 03／105／68 | 66.2593 | 03／29／68 | 24 | 1475 | Gross Alpha | 0.057 | 8．040E－02 | 63.0 | $19 / 019$ |
| $\bigcirc$ |  |  | 02／11／66 | 03／03／66 | 02123／68 | 66－2453 | 03／08／68 | 13 | 750 | Gross Apha | 0.024 | $3.820 \mathrm{E}-02$ | 21.0 | $6.5 / 0.065$ |
|  |  |  | 02／17768 | 03／08／66 | 02126／68 | 86－1892 | 03108／68 | 10 | 930 | Gross Apha | 0.021 | 2.800 E .02 | 17.0 | $52 / 0052$ |
| C |  |  | 01／18／66 | 02031／66 | 01／28／68 | 66－1820 | 0203／66 | 8 | 390 | Gross Alpha | 0.050 | 1．238E．02 | 350 | 11／0．31 |
| \％ |  |  | 02／24／66 | 03／19／66 | 03／07／66 | 66－2566 | 03／19／66 | 12 | 800 | Gross Atpha | 0.060 | 8．030E．02 | 530 | 18／0． 16 |
| $\bigcirc$ |  |  | 02／09／86 | 03／09／66 | 02123／68 | 66－2398 | 03／09／63 | 14 | 1200 | Gross Alpha | 0.028 | 7.150 E .02 | 250 | 7．710．077 |
| O |  |  | 02110166 | 03／19／60 | 02128／66 | 66－2046 | 03／19／68 | 19 | 695 | Gross Alpha | 0.013 | $8.480 \mathrm{E}-02$ | 13.0 | 410.04 |
| N |  |  | 02／06／66 | 03／08／66 | 02／21／66 | 66－4869 | 03／08／66 | 15 | 1020 | Gross Alpha | 0.050 | 1.250 E .02 | 470 | 14：0．14 |
| $\stackrel{\square}{0}$ |  |  | 01／18666 | 03／08／66 | 02／11／66 | 66－2623 | 03／0866 | 25 | 1100 | Gross Apha | 0.070 | $7.900 \mathrm{E} \cdot 02$ | 770 | $24 / 024$ |
|  |  |  | 01／1866 | 01／22166 | 01／20166 | 66－1138 | 01／22／68 | 2 | 975 | Gross Alpha | 0.057 | 2.560 E .02 | 7.9 | $24 / 0.024$ |
|  |  |  | 02／26／66 | 03／18／68 | 031／08／66 | 66－2112 | 03／18／68 | 10 | 580 | Gross Alpha | 0.058 | 6．960E．01 | 470 | 14／0． 14 |
| $\bigcirc$ |  |  |  |  |  | 66－1157 | 02／15／86 |  | 645 | Gross Alpha | no | ND |  |  |
|  |  |  |  |  |  | 66.3128 | 04／13／66 |  | 1650 | Gross Alpha | NR | NR |  |  |
| O1 | （10）（6） | （b）（6） | 01／18／66 | 03／28／66 | 02／21／66 | 66－2681 | 03／28／68 | 35 | 1520 | Gross Apha | 0.061 | 7．140E．02 | 750 | 23／0 23 |




NR - Not Reported, ND - No Detectabla Activily: N/a - not available

|  | Results of Modeling Individuals with Samples Assumed to be＂Uncontaminated．＂ |  |  |  |  |  |  |  |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
|  | NAME | SSN | $\begin{gathered} \text { START } \\ \text { EXPOSURE } \\ \text { DATE } \\ \hline \end{gathered}$ | $\begin{gathered} \text { END EXPOSURE } \\ \text { DATE } \end{gathered}$ | estimated <br> ACUTE <br> EXPOSURE <br> DATE | SAMPLE\＃ | SAMPLE | $\begin{gathered} \text { ELAPSED } \\ \text { DAYS } \end{gathered}$ | SAMPLE VOLUME （ mL ） | ANALYSIS | Cell shaded＝Additonal cempla for indiv not modelod slica $>0.1 \mathrm{pCi} / \mathrm{sampla}$ ，NR，or ND |  |  | $\begin{gathered} \text { 50-YEAR } \\ \text { CEDE } \\ (\text { ram } / \mathrm{SV}) \\ \hline \end{gathered}$ |
|  |  |  |  |  |  |  |  |  |  |  | $\begin{aligned} & \text { RESULT } \\ & (\rho C I I \\ & \text { samplo) } \end{aligned}$ | $\begin{gathered} +1-(p \mathrm{p} C \\| \\ \text { samplo }) \end{gathered}$ | $\begin{gathered} \text { INTAKE (1000s } \\ \text { pCi) } \end{gathered}$ |  |
| － | （10）（6） | （b）（6） | 01／17／186 | 02／25／68 | 02105／66 | 68－1341 | 02／25／68 | 20 | 600 | Gross Apha | 0.030 | $2.330 \mathrm{E}-02$ | 31.0 | $95 / 0095$ |
| ¢ |  |  | 02／10／66 | 03／04／68 | 02／21／66 | 66－2376 | 03／04／66 | 11 | 220 | Gross Alpha | 0004 | 1．010E．01 | 36 | $11 / 011$ |
| 3 |  |  | 01／18／68 | 01／29166 | 01／23／66 | 66－1077 | 01／29／68 | 6 | 1415 | Pu239 | 0079 | 1．975E－02 | 420 | 13／0．13 |
| （1） |  |  | 01／17／66 | 01／18／66 | 01／17／66 | 66－1134 | 01／18／68 | 1 | $\mathrm{n}^{1}$ | Gross Alpha | 0034 | 2 200E． 02 | 2.9 | 089100089 |
| あ |  |  | 01／18／68 | 02／03／66 | 01／26／66 | 60－1834 | 02／03／66 | B | 280 | Gooss Alpha | 0018 | 4．500E－03 | 130 | 4.004 |
| 7 |  |  | 02／18／68 | 03／04／83 | 02／25166 | 66－2342 | 03／04／65 | 7 | 900 | Gross Alpha | 0.092 | 1．060E－01 | 580 | 18／0． 18 |
| ${ }_{0}$ |  |  | 03／12／68 | 03／19／66 | 03／15／66 | 66－2189 | 03／19／63 | 4 | 1000 | Gross Alpha | 0009 | 2 250E－03 | 28 | $086 / 00086$ |
| 군 |  |  | 01／18／66 | 01／20166 | 01／19／68 | 66－1128 | 01／20／63 | 1 | 800 | Gross Alpha | 0028 | 2．120E－02 | 2.4 | 0.74100074 |
| ¢ |  |  | 02／21／66 | 03／19／68 | 03／00／66 | 66－2330 | 03／19／65 | 13 | 920 | Gross Atpha | 0047 | 8．110E－02 | 430 | 13／0． 13 |
| c |  |  | 01／18／68 | 01／29166 | 01／23／68 | 66－1070 | 01／29／68 | 8 | 2200 | Pu239 | 0.035 | 8．750E．03 | 19.0 | 5.80058 |
| 呂 |  |  | 01／29／66 | 03／18／66 | 02122／66 | 66－2099 | 03／18／69 | 24 | 870 | Gross Alpha | 0.003 | 8 270E－02 | 3.3 | 1／0．01 |
| ¢ |  |  | 02113／68 | 03／08／68 | 0224／66 | 66－2426 | 03／08／68 | 12 | 1000 | Gross Alpha | 0.034 | 1．890E－02 | 300 | 920092 |
| 5 |  |  | 01／21／66 | 03／08／66 | 02／13／68 | 66－2449 | 03／08／63 | 23 | 600 | Gross Alpha | 0034 | 1．990E－02 | 37.0 | 1／10． 11 |
|  |  |  | 03／01／66 | 03／08／66 | 03／04／66 | 60－2450 | 03／08／65 | 4 | 1150 | Gross Apha | 0.072 | 2．510E－02 | 22.0 | 6.810068 |
| 0 |  |  | 02110／68 | 03／08／68 | 02123／66 | 66－2447 | 03／08／63 | 13 | 1300 | Gross Apha | 0026 | 1660E－02 | 24.0 | 7.410074 |
| ¢． |  |  | 02106／66 | 02／28／66 | 0211766 | 66－1369 | 0212866 | 11 | 960 （12－ht） | Gross Alpha | 0018 | 4．500E．03 | 310 | $95 / 0095$ |
| 0 |  |  | 03／15／6E | 03／19／68 | 03／17／66 | 66－2570 | 03／19／65 | 2 | 700 | Gross Alphe | 0084 | 9．520E－02 | 120 | 3710037 |
| en |  |  | 03／00666 | 03／19／66 | 03／13／66 | 66－2325 | 03／19／63 | 6 | 910 | Gross Alpha | 0082 | $9.760 \mathrm{E}-02$ | 330 | 100.1 |
| O |  |  | 03／16／66 | 03／19／68 | 03／17／86 | 66－2556 | 03／19／65 | 2 | 950 | Gross Apha | 0045 | 8．760E－02 | B 1 | 1818019 |
| 考 |  |  | 02／23／66 | 03／08166 | 03／01／66 | 66－2509 | 03／08／63 | 7 | 900 | Gross Alpha | 0.090 | 2 250E－02 | 570 | 18／0．18 |
| 0 |  |  | 01／12／66 | 01129／68 | 01／23／66 | 66－1073 | 01／29／63 | 6 | 510 | Pu239 | 0.097 | 2．425E－02 | 51.0 | 16／0．16 |
| ＜ |  |  | 02／27／66 | 03108／66 | 03103／66 | 66－2456 | 03／00／65 | 5 | 700 | Gross Apha | 0．098 | $2570 \mathrm{E}-02$ | 410 | 13／0．13 |
| 0 |  |  | 02／27766 | 03／08／66 | 031／03／66 | 66－2465 | 03／00／68 | 5 | 550 | Gross Alpho | 0.072 | $2350 \mathrm{E} \cdot 02$ | 300 | 020.092 |
|  |  |  | 01／17／66 | 01／18／66 | 0117766 | 66－1138 | 01／11／68 | 1 | 725 | Gross Aphe | 0.068 | 2．490E－02 | 58 | 1．8\％0．018 |
| $\stackrel{\square}{2}$ |  |  | 02／25／66 | 03／18／66 | 03／07／66 | 66－2237 | 03／18／68 | 11 | 340 | Gross Aphe | 0.062 | 9．660E－02 | 530 | 16／0．16 |
|  |  |  |  |  |  | 68－772 | 02／11／63 |  | 360 | Gross Alpha | 1 | 4．000E－01 |  |  |
| c | （b）（6） | （b）（6） | 0272766 | 03／16／66 | 03／07／66 | 66－2670 | 03／27／68 | 20 | 820 | Gross Alpha | 0086 | 9．510E－02 | 68.0 | 21／0．21 |
| is |  |  | 02／17／68 | 03108／66 | 02126／66 | 66－1894 | 03／08／68 | 10 | 900 | Gross Apha | 0075 | $2380 \mathrm{E}-02$ | 62.0 | 19／0．19 |
| $\bigcirc$ |  |  | 02／25166 | 03／19／86 | 03／08／68 | 88－2228 | 03／19／63 | 11 | 920 | Gross Alpha | 0002 | 1060E－01 | 15 | 046／0．0046 |
| 0 |  |  | 0222766 | 03／19／66 | 03099／66 | 66－2313 | 03／19／68 | 10 | 1350 | Gross Alpha | 0.061 | 8．870E－02 | 50.0 | 15／0．15 |
| N |  |  | 02／10166 | 03／08／66 | 02／23／66 | 88－2451 | 03／08／68 | 13 | 1250 | Gross Atphe | 0012 | 1420E－02 | 11.0 | 3．4／0．034 |
| （1） |  |  | 02／13／66 | 03／23／66 | 03／04／66 | 66－2583 | 03／23／68 | 19 | 1600 | Gross Alpha | 0.060 | 8．030E．02 | 610 | 19／0．19 |
|  |  |  | 02／25／66 | 03／19／68 | 03／08／68 | 66－2225 | 03／19／68 | 11 | 1000 | Gross Apha | 0037 | 9．770E－02 | 320 | $98 / 8.098$ |
|  |  |  | 01／25／66 | 03／13／66 | 0217168 | 66－2547 | 03／13／68 | 24 | 750 | Gross Atpha | 0.098 | 1．010E－01 | 1100 | 34／0．34 |
| $\bigcirc$ |  |  | 02／13／66 | 03／08／66 | 0224／66 | 88－2467 | 03／00／68 | 12 | 1000 | Gross Atpha | 0.032 | 1．970E－02 | 29.0 | $89 / 0.089$ |
| $\infty$ |  |  | 01／18／66 | 02／03／66 | 01／26／66 | 66－1825 | 0203166 | 8 | 600 | Gross Apha | 0050 | 1238E－02 | 350 | 11／0．11 |
|  |  |  | 02／13／66 | 03119／66 | 03／02166 | 66－2182 | 03／19／63 | 17 | 900 | Gross Alpha | 0.072 | 1．800E－02 | 71.0 | 2210.22 |




NR - Not Reported; ND - No Detectable Aclivity; n/a - not available

| (1) | Results of Modeling Indivlduals with Samples Assumed to be "Uncontaminated." |  |  |  |  |  |  |  |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| $\begin{aligned} & \overline{\overline{0}} \\ & \underset{\sim}{\omega} \\ & \underset{\sim}{2} \end{aligned}$ |  |  |  |  |  |  |  |  |  |  | Cell shad modeled | Addilonal 3 sa ca $>0.1 \mathrm{pC} / 3 a$ <br> $\mathrm{Ce}_{\mathrm{e}}>0.1 \mathrm{pCl} / 3$ | pla for indiv not ple, NR, or ND |  |
| $\begin{gathered} 0 \\ \frac{0}{n} \\ \frac{3}{\omega} \end{gathered}$ | NAME | SSN | $\begin{gathered} \text { START } \\ \text { EXPOSUE } \\ \text { DATE } \\ \hline \end{gathered}$ | $\begin{gathered} \text { END EXPOSURE } \\ \text { DATE } \end{gathered}$ | $\begin{gathered} \text { EsTIMATED } \\ \text { ACUTE } \\ \text { EXPOSUUE } \\ \text { DATE } \end{gathered}$ | SAMPLE\# | $\begin{gathered} \text { SAMPLE } \\ \text { DATE } \\ \hline \end{gathered}$ | $\begin{gathered} \text { ELAPSED } \\ \text { DAYS } \\ \hline \end{gathered}$ | SAMPLE <br> VOLUME <br> ( mL ) | ANALYSIS | $\begin{gathered} \text { RESULT } \\ \text { (pCII } \\ \text { samplo) } \\ \hline \end{gathered}$ | $\begin{aligned} & \text { tr ( } \mathrm{p} \mathrm{C} / l \\ & \text { samplo) } \end{aligned}$ | $\begin{gathered} \text { INTAKE (1000s } \\ p C i) \end{gathered}$ | S0-YEAR CEDE $($ rem/Sv $)$ |
| 응 | (b) (6) | (b) (6) | 01/18/66 | 01/29/66 | 01/23/66 | 66-1083 | 01129/66 | 6 | 690 | Pu239 | 0.062 | 1550 E -02 | 330 | 10101 |
| $\bigcirc$ |  |  | 02/18/66 | 03/08/66 | 0222786 | 66-2468 | 03108/68 | 9 | 450 | Gross Apha | 0.090 | $2320 \mathrm{E} \cdot 02$ | 69.0 | 210.21 |
|  |  |  | 03/26/66 | 04/10/66 | 04/02/66 | 66-3206 | 04/21/68 | 19 | 875 (12-ht) | Gross Apha | 0.076 | 8240E-02 | 150.0 | 46/0.46 |
| (1) |  |  | 0209/66 | - 02/23/66 | 0211866 | 68-1362 | 0228/66 | 10 | 675 (12-ht) | Gross Apha | 0.041 | 1.013E-02 | 66.0 | 20/0.2 |
| क |  |  | 0211166 | 03/19/66 | 03101/66 | 66-2169 | 03/19/66 | 18 | 650 | Gross Apha | 0.047 | 1.163E-02 | 47.0 | 14/0.14 |
| \% |  |  | 01/22166 | 03/13/66 | 0216/66 | 66-2541 | 03/13/68 | 25 | 1400 | Gross Apha | 0.059 | 1.463E-02 | 650 | 2000.2 |
| 8 |  |  | 02/17/6E | 03/03/68 | 02288/68 | 68-1890 | 03308/66 | 10 | 890 | Gross Apha | 0.087 | 3.460E-02 | 71.0 | 22\%.22 |
| $\stackrel{\square}{\square}$ |  |  | 01/18/66 | 01/29/66 | 01/23/66 | 66-1076 | 01/29/66 | 6 | 510 | Pu239 | 0.079 | 1.975E-02 | 42.0 | 13/0. 13 |
| \% |  |  | 02/25/66 | 03/19/66 | 03108/68 | 68-2203 | 0319/66 | 11 | 950 | Gross Apha | 0.032 | 7.875E-03 | 27.0 | 8310.083 |
| c |  |  | 02/21/66 | 03/18/86 | 0305/63 | 66.2255 | 03/18/68 | 13 | 720 | Gross Apha | 0.081 | 1.022E-01 | 73.0 | 220.22 |
| a |  |  | 01/18/66 | 03/19/66 | 02117/68 | 66-2042 | 03119/66 | 30 | 300 | Gross Apha | 0.076 | 7.120E.02 | 89.0 | 2710.27 |
| 9 |  |  | 02/13/66 | 03146/66 | 02128163 | 66-2029 | 03/16/66 | 16 | 620 | Gross Apha | 0.095 | 8.590E-02 | 92.0 | 28/0.28 |
| 耍 |  |  | 0209/66 | 03/04/66 | 02/20/65 | 66-2382 | 03304166 | 12 | 600 | Gross Apha | 0.062 | 8.760E-02 | 550 | 1710.17 |
| $\bigcirc$ |  |  | 01/18/68 | 01/21/65 | 01/19/66 | 68-1127 | 01/21/68 | 2 | 550 | Gross Apha | 0.064 | 2.270E-02 | 8.7 | $27 / 10.027$ |
| $\bigcirc$ |  |  | 02105/66 | 03/08/66 | 0220/66 | 66-1888 | 03/08/66 | 16 | 810 | Gross Apha | 0.037 | 2.010 E .02 | 35.0 | 11/0.11 |
| क. |  |  | 0224/66 | 03/08/66 | 0310266 | 66-1884 | 03/08/66 | 6 | 600 | Gross Apha | 0.018 | 2.010 E .02 | 9.4 | $29 \% 0.029$ |
| O. |  |  | 03/14/66 | 03/19/66 | 03/46/66 | 66-2138 | 03/19/66 | 3 | 550 | Gross Apha | 0.066 | 8.040E.02 | 140 | 4310.043 |
| 0 |  |  | 02/10/66 | 03/08/66 | 0223/66 | 66-2435 | 03/08/68 | 13 | 400 | Gross Apha | 0.024 | 1.390E-02 | 220 | $68 \% .068$ |
| $\bigcirc$ |  |  | 02110/68 | 03108/66 | 0222366 | 66-2432 | 03308/66 | 13 | 1000 | Gross Apha | 0.013 | 2.170E-02 | 11.0 | 34/0.034 |
| \% |  |  | 02/10/68 | 03/19/66 | 0228/63 | 66-2185 | 03/19/68 | 19 | 400 | Gross Apha | 0.059 | 1.463E-02 | 590 | 18/0.18 |
| 0 |  |  | 02/13/68 | 03/18/66 | 03101166 | 66-2250 | 03/18/68 | 17 | 830 | Gross Apha | 0005 | $9047 \mathrm{E} \cdot 02$ | 5.0 | 15/0.015 |
| 근. |  |  | 0206/66 | 02128/66 | 0217766 | 66-1374 | 02/28/63 | 11 | 930 (12-hr) | Gross Apha | 0.072 | 1 103E-02 | 120.0 | 37/0.37 |
| 0 |  |  | 02/27/68 | 03/19/66 | 03109166 | 66-2321 | 03/19/68 | 10 | 550 | Gross Apha | 0.010 | 5.176E-02 | 8.6 | 2610.026 |
| $\bigcirc$ |  |  | 01/18166 | 02103/66 | 01/26/63 | 66.1824 | 02/03/68 | 8 | 760 | Gross Apha | 0.018 | 4500 E -03 | 130 | 4/0.04 |
| $\stackrel{\square}{\circ}$ |  |  | 01/24/66 | 03/04/66 | 0212/68 | 66-2374 | 03/04/6G | 20 | 1300 | Gross Apha | 0.026 | $7150 \mathrm{E}-02$ | 270 | $83 / 0083$ |
| Cl |  |  | 01/18166 | 0203/66 | $01 / 26166$ | 66-1818 | 02/03/68 | 8 | 930 | Gross Apha | 0.041 | $1.013 \mathrm{E}-02$ | 290 | 8 890.089 |
| C |  |  | 01/18/66 | 02003/66 | 01/26/66 | 66-1828 | 02003/66 | 8 | 300 | Gross Apha | 0099 | 2478E.02 | 70.0 | 220.22 |
| ¢ |  |  |  |  |  | 66.2400 | 03109/68 |  | 350 | Gross Apha | 0.343 | 1850E-01 |  |  |
| $\bigcirc$ | (b) (6) | (13) (6) | 01/18/66 | 03/13/66 | 02144/66 | 66-2540 | 03/13/68 | 27 | 400 | Gross Apha | 0.050 | 1238E.02 | 56.0 | 177.17 |
| ¢ |  |  | 01/31/66 | 03/18/66 | 02123/66 | 66-2272 | 03/18/66 | 23 | 300 | Gross Alpha | 0.083 | 1.029E-01 | 900 | 28/0.28 |
| N |  |  | 02/10/66 | 03/09/68 | 0223166 | 66-2441 | 03008/66 | 13 | 950 | Gross Apha | 0.004 | 1642E-02 | 38 | $12 / 0.012$ |
| © |  |  | 02/13/66 | 03/19/66 | 03/02166 | 66-2222 | 03/19/66 | 17 | 900 | Gross Alpha | 0.066 | 8 046E-02 | 65.0 | 200.2 |
|  |  |  | 02/13/66 | 03/19/66 | 03/0266 | 66-2197 | 03/19/68 | 17 | 600 | Gross Apha | 0.030 | $7500 \mathrm{E} \cdot 03$ | 30.0 | 920.092 |
| $\bigcirc$ |  |  | 0224/66 | 03/19/66 | 03107/66 | 66-2171 | 03/19/86 | 12 | 750 | Gross Apha | 0.081 | 2.025E.02 | 72.0 | 220.22 |
| $\omega$ |  |  | 02/03/66 | 02125/66 | 02141/66 | 66-1339 | 02/25/68 | 11 | 490 | Gross Alpha | 0.062 | 2.420E-02 | 53.0 | 16/0.16 |
| $\stackrel{\rightharpoonup}{ \pm}$ |  |  | 02/18/66 | 03/09/66 | 02/27/66 | 68-2054 | 03109/66 | 10 | 1480 | Gross Alpha | 0.075 | 7.120E-02 | 61.0 | 19/0.19 |
|  |  |  | 02\%eß66 | 03/08/66 | 0222/66 | 68-1881 | 03/08/66 | 14 | 900 | Gross Apha | 0.036 | 2.190E-02 | 330 | $10 / 0.1$ |

[^9]


## Appendix C. 4

## Remaining Cases

Portions of this report have been designated records subject to the restriction of the Privacy Act, 5 U.S.C. 552(a) and are so marked.

## APPENDIX C. 4 Remaining Cases

Most of those who responded to the Palomares Broken Arrow submitted one urine sample that was collected during their time on site at Camp Wilson, or nearby. Furthermore, these generally were collected with containers designed for other purposes and under conditions that provided only limited protection against contamination with plutonium in blowing dust. Also, analysis of most of the samples by the gross alpha counting method served primarily as a screening for further study. Since most of the responders were not identified for follow-up, their initial samples were their only sample.

A small number of those in this group initially qualified for assessment in the "Contamination Cutoff" Cases. However, the chemical recovery for the samples processed for alpha spectrometry did not meet the criterion established for this study. Therefore, the data for these individuals are reported in this Remaining Cases category.

Intake and dose assessments were not performed for the cases in this category because the data were considered unreliable. Possible sample contamination, laboratory contamination, and uncertain recording of collection information limit the usefulness of these data for assessing intake and dose. The urine results ranged from 0 to 237.9 pCi per sample. The latter sample, collected three days after the first airmen arrived at the accident site, represents a prime example of possible contamination. That sample was the only sample available for the individual concerned. Personal discussions with one of the first responders indicated that the initial samples were collected using wine, milk, and any other type of bottle available in the village. (Skaar 1999).

The following pages provide a listing of the results for the Remaining Cases. If evaluated, the results documented would produce intakes ranging from about $75,000 \mathrm{pCi}$ to $20,000,000 \mathrm{pCi}$
corresponding to CEDEs of about 23 rem to 6,000 rem ( 0.23 to 60 Sv ). Results of this magnitude require careful evaluation. The listings contain the basic sample identifying, collection and result information. Hardcopy laboratory records support each of the entries and are maintained by the Air Force.



[^10]



[^11]

| Individuals with Urine Samples Classified as Remaining Cases |  |  |  |  |  |  |  |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| Namo | 85N | Estimated Start Exposura Dato | Estimatod End Exposure Dato | Estimated Acuto Exposure Data | Sampla No. Sampla Dato |  | $\begin{gathered} \text { Samplo Volt } \\ \text { (mLl } \end{gathered}$ | Baso | Sampla <br> Anal | Data Anal | Rasult (pCl/sampla) | 24hr Activity〈pCliday) | Recorded <br> Systomic Body Burden |
| (b) (6) | (b) (G) | 01/20188 | 02109168 | 01/30/66 | ${ }^{66.820}$ | 0209168 | 880 | Toul Rosteres | 200 | N/A | $2.45+1 / 1.29$ | 4.253 | 0.0112 |
| (b) (6) | (b) (6) | 02109/60 | 03/09/68 | 0223/66 | -66-3264 | $08101 / 88$ $03 / 2966$ | 2010 420 | Toul Rosleres Torroion | 1160 420 | N/A | $\stackrel{0}{0}$ | 0.000 0.934 | 0.00427 |
|  |  | 0208186 | 03/01/86 | 02118/66 | 86-1851 | 03/01/68 | 750 | Moron | 750 | N/A | 0.443+/-0.176 | 0.709 |  |
|  |  |  |  | 02118/68 | 66.1851 | 03/01/86 | 750 |  |  | 07/11/86 | $0.48+1-0.020$ | 0.736 |  |
| (b) (6) | (b) (6) | $0205 / 88$ | 03/08/86 | 0220186 | 68.2481 | 03010168 | 900 | Litle Creek | 900 | N/A | $0.132+1-0.030$ | 0.176 |  |
|  |  | 01/18/88 | 02/10/68 | 01/29/88 | 68-489 | 02107168 | 310 | Moron | 200 | N/A | 7.32+1-0.87 | 28.335 | 0.0915 |
|  |  |  |  |  | 66-1442 | 03/02188 | 1920 | Moron | 1800 | 03/77138 | 0.432+/-0.27 | 0.432 | 0.00291 |
| (10) (6) | ) (6) | ${ }^{011 / 18 / 86}$ | 03177/66 | 02118/66 | ${ }^{866.2021}$ | 03177166 | 1400 | ${ }^{244 \mathrm{~h}} \mathrm{Avn}$ Bn | 1400 | N/A | ND | ND |  |
|  |  | 01118166 | 02/06/86 | $01 / 27168$ | 68.511 | 02028186 | 920 | Torelon | 200 | N/A | 1.46+\|-1.32 | 1.904 | 0.00584 |
|  |  | 01/25/186 $02 / 0 / 186$ | 02090136 $02126 / 66$ | 02/01/66 $02115 / 86$ | 66-794 | 0209168 | 930 820 | Hanaw, Gormany | 200 852 | N/A | ${ }^{1.80+1-1.24}$ | 2.323 | 0.0079 |
|  |  |  |  |  | 16.1437 | 0228166 | 820 | Torrojon |  | 08/23/66 | ${ }_{1} .512+1-0.113$ | 0.347 2.213 |  |
| (15) (6) | (1) (6) | 02131/66 | ${ }^{03 / 191868}$ | 03/02166 | 66.2152 | 03/19/66 | 380 | Torrejon | 380 | N/A | $0.208+1 / 0.129$ | 0.857 | 0.00127 |
|  |  | 03311/86 | 03119198 | 03/15/86 | 66-2219 | 03/19166 | ${ }^{1050}$ | Torrejon | 1050 | N/A | 0.197+/-0.124 | 0.225 | 0.000958 |
|  |  | 03/28/86 | 04/10/86 | 0403/66 | 66-2990 | 04/26/66 | 1875 | Furth, Ger. US | 1875 | N/A | ND | ND |  |
|  |  |  |  |  |  |  |  | Army, 201h Sta Hosp. |  |  |  |  |  |
| (b) (6) | (b) (6) | 0209186 | 03108/86 | 02122166 | 66.1875 | 03/08/66 | 560 | Moron | 582 | N/A | 0.958 | 2.053 | 0.0049 |
|  |  | 01118/86 | 03/08/86 | 02/11/66 | 66.2618 | 03108/66 | 1750 | Vandenburg | 1750 | N/A | 0.319+/-0.156 | 0.319 |  |
|  |  | 01118/66 | 02/10166 | 01/29/68 | 68-768 | 02/11/66 | 440 | Torrejon | 200 | N/A | 0.52+/-0.48 | 1.418 | 0.00536 |
|  |  | 01/18/6E | 03/08/66 | 02411/66 | 66.2620 | 03/08/66 | 1350 | Vandenburg | 1350 | N/A | ND | No |  |
|  |  | 01/18/66 | 01/22768 | 01/20/86 | 88.1107 | 01/22/88 | 1170 | Pirmasens | 1000 | 03/17168 | ND | ND |  |
|  |  | 02108186 | 02/17/66 | 0212166 | 66-1113 | 02177166 | 960 | San Pablo | 960 | 03/23/66 | 0.539+/-0.251 | 0.874 | 0.00125 |
|  |  | 03114/68 | 03/19165 | 03/16/66 | 66-2154 | 03/19188 | 490 | Torrejon | 490 | N/A | 0.135+/-0.119 | 0.331 | 0.000196 |
|  |  | 01/18/88 | 04/11/68 | 02/28/66 | 86.2945 | 04/22/68 | 1000 | B097 | 1000 | N/A | ND | ND |  |
| (b) (6) | b) (6) | 01/18/68 | 02/08/68 | 0128/66 | ${ }_{56-2945}$ | 04/22/66 | 1000 |  |  | 08/01186 | NR | NR |  |
|  |  | 01/18/68 | 03/17/68 | 02/18/68 | 66-2020 | 020108766 | ${ }^{620} 1450$ | ${ }_{\text {24th }}^{\text {Mun }}$ Morn | ${ }_{1450}$ | N/A | $\stackrel{\text { ND }}{\substack{\text { N } \\ 0.137+10.079}}$ | ND 0.137 | 0.000948 |
|  |  | 03/01/66 | 04/11/68 | 03/21/66 | 86.3199 | 05/12186 | 890 | Torrejon | 890 | N/A | $0.186+/ 0.082$ | 0.251 | 0.00098 |
|  |  | 01/28/36 | 02/28/66 | 02112/66 | 66-2884 | 03/31/86 | 1750 | Zeragoza | 1750 | N/A | 0 | 0.000 |  |
|  |  |  |  |  | 66.2884 | 03/31/66 | 1750 |  |  | 08/17/86 | NR | NR |  |
| (b) (6) |  | 01/22186 | 04/11/68 | 03/02/68 | 86-2991 | 04/21/66 | 600 | Furth, Ger. US | 600 | N/A | ND | ND |  |
|  |  |  |  |  |  |  |  | Army, 20th Sta Hosp. Then |  |  |  |  |  |
| (0) (6) | 3) (6) | 01/18/68 | 02/04/36 | 01/26/68 | 66-350 | 0204/66 | 700 | Torrejon | 200 | N/A | ND | ND |  |
|  |  | 011/18168 | $0^{02108 / 368}$ | $01 / 27768$ | 68.501 | 02108186 | 440 | Maron | 200 | N/A | $2.43+1 / 0.67$ | 6.627 | 0.0202 |
|  |  | 01/18/66 | 03/03/68 | 02/09/66 | 66-1938 | 03/03/66 | 1700 | 625TH MASS (MAC) | 1780 | N/A | $2.20+1 / 0.29$ | 2.200 | 0.0215 |
| (b) (6) | ) (6 | $02 / 2766$ | 03/18/66 | 03/08/66 | 66-2285 | 03/18/86 | 780 | Torrejon | 780 | N/A | 0.173+/-0.130 | 0.266 | 0.00101 |
|  | ) | 01/291/68 | 02/19/68 | 02008/88 | 68.1219 | 02191986 | 940 | Torrejon | 940 | N/A | ND | ND |  |
|  |  | 03/14/66 | 03/19168 | 03/16/66 | ${ }^{66-2582}$ | 03/19/66 | ram | Torrejon | n/a | N/A | NR | NR |  |
| ) (6) | ) (6) | 01/18/86 | 04/11/186 | 02/28168 | 66.3108 | 04/13/66 | 1900 | Maron | 950 | N/A | 0.364+/-0.135 | 0.364 |  |
|  |  | 0200168 | 03/08/68 | 02221/68 | 66-1883 | 03/08/86 | 720 | Torrejon | 720 | N/A | 0.120+1-0.030 | 0.200 | 0.000669 |
|  |  | $0204 / 66$ | 03/18/86 | 02225166 | 86.2092 | 03141866 | 780 | USS FL. Snelling | 780 | N/A | 0.261+/-0.109 | 0.402 | 0.00187 |
|  |  | 03/14/86 | 03/20/66 | 03/17/66 | 86.2301 | 03/20168 | 750 | Torejon | 750 | N/A | $0.129+1-0.131$ | 0.206 |  |
|  |  | 01/18/68 | 02109/68 | 01129/68 | 66-817 | 02099/66 | 680 | Gemmany | 200 | N/A | ND | ND |  |
|  |  | 01/19166 | 02/19/68 | 0203/66 | 66-2881 | 03/31/86 | 900 | Tomejon | 900 | N/A | 1.88 +1/0.33 | 2.240 |  |
|  |  |  |  |  | 66.2881 | 03/31/66 | 900 |  |  | 08/23/68 | NR | NR |  |
| (b) (6) | (1a) (6) | 01/21/86 | 02/28/66 | 02/08/68 | 66-1439 | ${ }^{02228 / 68}$ | 760 | Torrejon | 790 | N/A | ND | ND |  |
| (b) (6) | (b) (6) |  |  |  | 66-1439 $88-2350$ | $022 / 12186$ $03 / 04 / 66$ | 760 600 |  |  | 08/23/68 | $0.678+1 / 0.107$ ND | ${ }^{1.071}$ |  |
|  |  | 02117/68 | 02255/68 | 02205/65 | 66-1351 | 02125/66 | 720 | Moron | 748 | N/A | $0.410+1 / 0.588$ | 0.683 |  |
|  |  | 0131788 | 0 018188 | 0223366 | 86:2115 | $031 \mathrm{~B} / 66$ | 890 | Giasson | 890 | N/A | 6.13777.0.096 | 0.77 | 0.06099 |
|  |  | 01/181868 | 02118/60 | 0201186 | 66-980 | 02/16/86 | 760 | Torrejon | 200 | N/A | NR | NR |  |
|  |  | 02/18/86 | 03/19/66 | 03/04/66 | 66.2189 | 03/199/6B | 800 | Torrejon | 800 | N/A | 0.202 | 0.303 |  |
|  |  | 01/25/86 | 02009/66 | 0201/68 | 66-734 | 02091/66 | 510 | Hanaw, Garmany | 200 | N/A | 0.78+/-0.55 | 1.835 | 0.0063 |
|  |  | 02206686 | 02108/66 | 02077168 | 68-1381 | 0228186 | 330 (12-hr) | Moron | 343 | N/A | 1.32 | 1.320 | 0.005 |
|  |  | 0204/66 | 03/03/68 | 0217166 | 68-1912 | 03103/68 | 900 | USS Charieston | 900 | N/A | $0.1999+$-0.089 | 0.265 |  |
|  |  | 022/25/66 | 03/17/68 | 03/07/68 | 68-2700 | 03/30/66 | 950 | Moron | 950 | N/A | $0.158+/ \cdot 0.113$ | 0.200 |  |

[^12]


[^13]

Palomares Nuclear Weapons Accident DRAFT




Palomares Nuclear Weapons Accident DRAFT
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| Individuals with Urine Samples Classified as Remalning Cases |  |  |  |  |  |  |  |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| Name | SSN | $\begin{aligned} & \text { Estimatod } \\ & \text { Start } \\ & \text { Exposure } \\ & \text { Dato } \\ & \hline \end{aligned}$ | Estumated End Exposuro Dato |  | Sample No. Sampla Date |  | $\begin{gathered} \text { Sample Vol } \\ (\mathrm{mL}) \end{gathered}$ | Baso | Samplo Anal | Dato Anal | Result (pCl/samplo) | 24hr Actlulty (pCl/day) | Recorded <br> Systomle Bady Burdan |
| 0) (6) | (b) (6) | 02/11/66 $01 / 25 / 86$ | $03 / 19166$ $0221 / 68$ | 03/01/66 02077168 | $66-2132$ $68-1114$ | $03 / 19 / 68$ $01 / 24 / 68$ |  | Tormejon | 880 | N/A | 0.212+1.0.134 | 0.289 | 0.00133 |
|  |  |  |  |  | $66-1114$ $66-1013$ | $01 / 24 / 68$ $0221 / 88$ | 980 1870 | Ramsietn | 980 | 03/23/68 | NR | NR |  |
| (6) (6) |  |  |  |  | 66-2583 | 03/14/66 | 2325 | Ramsieln | 2325 | N/A | $0.120++-0.108$ | ND 0.120 |  |
|  | D) (6) | 01/18/66 | 02/00/66 | 01/20/68 | 86-368 | $0204 / 68$ | 900 | Torrejon | 200 | N/A | $61.4+/ 4.3$ | 81.867 | 0.307 |
| (b) (6) | (b) (6) | 01/18/66 | 01/21/68 | 01/19/66 | 66-1180 $66-238$ | 02118/68 | 2120 500 | Torejon | 2120 | N/A | ND | ND |  |
| (b) (6) |  |  |  |  | -68-239 | $01 / 21786$ | 400 | Torrejon | 200 | N/A | NR | NR |  |
|  | (b) (6) | 01/18166 | 0207665 | 01/28/68 | 66-460 | $02107 / 68$ | 395 | Momen | 200 | N/A | $\stackrel{\text { 2.08 }}{\text { NR }+-0.71}$ | ¢ ${ }_{\text {NR }}$ |  |
|  |  | 01/18/88 | 03/19/86 | 02181/68 | 66-2244 | 03/18/68 | 440 | Torejon | 440 | N/A | ND | ND | 0.0202 |
|  |  | 01/18/68 | 0204/66 | 01/28166 | 66-339 | 02104188 | 900 | Torejon | 200 | N/A | $2.22+1 \cdot 0.99$ | 2.960 | 0.0111 |
|  |  | 02113/66 | 03/19/68 | 0310266 | ${ }^{86-2153}$ | 03/19168 | 940 | Torrejon | 940 | N/A | $0.137+1 \cdot 0.107$ | 0.175 | 0.000839 |
|  |  | 01/18168 | 0207/68 | $01128 / 68$ | ${ }^{66-465}$ | 0207188 | 960 | Torejon | 200 | N/A | $1.23+1 \cdot 0.82$ | 1.538 | 0.00469 |
|  |  | 01/18/66 | 0303/86 | 02/09/60 | 86-1936 | $031031 / 66$ | 860 1825 | Torrelon | ${ }_{1}^{2005}$ | N/A | ND | ND |  |
| (b) (6) |  |  |  |  |  |  |  | (MAC) |  | N/A | 0.225+/-0.118 | 0.225 |  |
|  | (6) | 02081/66 | 02/25166 | 02116/66 | 68-1354 | ${ }^{02125 / 66}$ | 500 | Toul Rosieres | 520 | N/A | ND | ND |  |
|  |  | 01/29/66 01/18/60 | $03 / 19166$ $02 / 26166$ | 0222166 | 66-2044 | 03/19/66 | 790 | Torrejon | 790 | 04/08/68 | NR | NR |  |
| (b) (6) |  |  |  | 0200/66 | 66-1414 | -02126/36 | 880 880 | Torrejon | 915 | N/A | ND | ND |  |
|  | ) (6) | 01/18186 | 01/21/36 | 01/19/68 | 66-265 | 0121/66 | 1100 | Torrejon | 200 | N/A | NR | NR |  |
|  |  | 01/18/66 | 0131/66 | 01/24/86 | ${ }^{66-381}$ | 0201/68 | 2800 | Sembach | 200 | N/A | ND | ND |  |
| (6) (6) | (0) (6) | 01/18/68 | 03/22/36 | 02/08/68 | ${ }_{66.912}^{66-381}$ | $0211 / 166$ | 2800 |  |  | n/a | ND | ND |  |
| (b) (6) | 6 |  |  |  | 88-1298 | 03102/86 | 2910 | Seymour Johnson | 1000 | N/A | ND | ND |  |
|  |  | 01/31/66 | 0320186 | 02/24/66 | 66-2291 | ${ }^{\text {c }} 31201688$ | 655 | Toul Rosleres Norithe | 200 | N/A | 0.64+1/0.52 | 1.173 | 0.00375 |
|  |  | 011118166 | 0222/66 | 02104/66 | 66-1243 | c2/22/66 | 810 | Offutt AFB, NE | 842 | N/A | 0.133+-0. ND | 0.290 ND | 0.00124 |
|  |  | 02/08/66 | 02/26/86 | 0216/66 | ${ }^{66-1423}$ | 02266/66 | 760 | Moron | 790 | N/A | 0.453+/0.175 | 0.715 |  |
| (10) (6) | (b) (6) | 01/18/66 |  | 02/01/66 | 66.1423 68.967 | 02/20/66 | 760 |  |  | 06/23/68 | 0.057+/-0.023 | 0.090 |  |
|  |  | 02/18166 | 03/18/86 | 03/04/66 | 66-2114 | 03/18/86 | 430 | Torrejon Moron | 200 | N/A | ND | ND |  |
|  |  | 02/08/88 | 02281/66 | 0216/66 | 66-1433 | 02261/66 | 720 | Moron | 748 | N/A | $0.131+/-0.178$ | ${ }_{0}^{0.500}$ | 0.00137 |
|  |  | 0210160 | 03/08/68 | 02/23/86 | 66-2510 | 03/08/166 | 1000 | Maron | 1000 | N/A | 0.45 | 0.540 |  |
|  |  | 01/18/66 | 03/25/88 | 02/20/66 | 60-2597 | 03/25/86 | 950 | USAH Wurzurg. | 950 | NIA | $0.371+/-0.210$ | 0.469 |  |
| (b) (6) | (b) (6) | 02/27186 | 03/19/88 | 03/09/88 | 66-2125 | 03/19/66 | 940 | Torrejon | 940 | N/A | ND | ND |  |
|  |  | 02105/86 | 02125/86 | 02115/86 | 60-1349 | 0225/66 | 600 | Sombach | 624 | NIA | $0.176+/-0.574$ | 0.352 |  |
|  |  | -0209/68 | 03/01/88 | 02199/66 | 66-1852 | 03/01/66 | 600 | San Pablo | 800 | N/A | 1.86 + -0. 35 | 3.720 |  |
|  |  | -02/25/66 | 031/1/768 | 03/07/86 | ${ }_{686}^{66-2284}$ | 03111/86 | 950 | Moron | 950 | N/A | $0.181+/-0.113$ | 0.203 | 0.000986 |
|  |  | 0202/86 | 0212260 | 02/12/66 | 66.1088 | 02222186 | 770 370 | San Pablo | 370 | N/A | $2.77+1-0.96$ | 4.234 | 0.0166 |
| b) (6) | ) (6) |  |  |  | 66.2592 | 03/28/66 | 1750 | San Pablo | 1750 | N/A | $0.229+1 / 0.184$ | 0.229 |  |
|  |  | 02121/88 | 03/19/68 | 03/04/86 | ${ }^{66}$-2048 | 03119168 | 800 | Moron | 800 | NIA | 0.269 $/$ /0. 109 | 0.404 |  |
|  |  | 0214888 | 03/12166 | 03/01/68 | ${ }_{66 \text { 6-2033 }}$ | 03/18168 | 1000 | Torrejon | 1000 | N/A | . $398+1 / 0.175$ | 0.475 |  |
|  |  | 02106166 | 0208166 | 0207168 | 86.1397 | 0228/68 | 880 (12-hr) | Moron | 915 | N/A | $0.167+1-0.093$ 0.195 | 0.167 0.195 |  |
|  |  | 01141/66 | 03/08/66 | $02111 / 86$ | 68.2477 | 03/08/66 | 250 | Torrelon | 250 | N/A | 0.38 | 1.728 | 0.0008 |
|  |  | 01118/66 | 020808168 | 01/27/86 | 68.502 | 02106/06 | 860 | Torrejon | 200 | N/A | 2.36+1-0.99 | 3.293 | 0.01 |
|  |  | $01 / 29 / 68$ $03 / 14 / 66$ | 02/22/68 | 02/10/88 $031 / 8 / 68$ | 66.1090 66.2159 | 02/22/88 | 1460 | Torrejon | 1460 | 031/7/86 | $4.35+1-0.53$ | 4350 | 0.0212 |
|  |  | 03/21/88 | 04/11/66 | 03/31/86 | 66-3198 | 03/19/66 | ${ }_{9} 900$ | Torrajon | 900 | N/A | $0.148+/-0.135$ | 0.197 | 0.000215 |
|  |  | 01/11/86 | 02001/6 | 01/27/66 | 60-503 | 0206/66 | 900 | Torrojon | 950 200 | N/A | ${ }_{\text {0 }}^{0.191+1+-0.089}$ | 0.241 |  |
|  |  | 0227166 | 03/24/88 | 03/11/66 | 66-2698 | 09/05/68 | 800 | Moron | 800 | N/A | $0.133+1-0.113$ | ${ }_{0}^{6.200}$ | 1.99 |
|  |  | $01 / 7716$ | 0202266 | 01/25/66 | 68-3211 | 05118/66 | 450 | 67ih ARS Prestwick MOA, Scolland | 450 | N/A | $0.161+1-0.078$ | 0.429 |  |
| (b) (6) | b) (6) | 02/11/68 | 0215/8B | 02/13/66 | 66.3208 | 0420/66 | 1340 | Torrelon | 670 | N/A | 0.500+/-0.198 | 0.500 |  |



|  |  | Individ | uals with Urin | Samples | Classified a | S Remaining | Cases |  |  |  |  |  |  |
| :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: | :---: |
| $\begin{aligned} & \text { Name } \\ & (b)(6) \end{aligned}$ | SSN | $\begin{gathered} \text { Estumated } \\ \text { Start } \\ \text { Exposure } \\ \text { Date } \\ \hline \end{gathered}$ | Estimatad End Exposura Dato | Estimated Acuta Exposura Dato | Samplo No. | Sample Dato | $\begin{gathered} \text { Somplo Vol } \\ \text { (mL) } \end{gathered}$ | Basa | $\begin{aligned} & \text { Sampla } \\ & \text { Anal } \end{aligned}$ | Date Anal | $\begin{gathered} \text { Result } \\ \text { (pCl/samplo) } \end{gathered}$ | 24hr Aetivity (pCl/day) | Recorded Systamic Body Burden |
|  | (5) (6) | $01 / 181 / 88$ 020968 | 04/11/86 $03 / 09 / 66$ | 02281896 $02123 / 68$ | ${ }_{6}^{66-3208-5}$ |  | 1340 1000 | $\stackrel{\text { N/a }}{\text { Torelon }}$ | 670 1000 | N/A | ${ }_{0}^{\text {a }}$ NR $136+1.0082$ | NR |  |
|  |  | 02109/68 | 03/09/66 | ${ }^{02123 / 66}$ | ${ }_{\text {cher }}^{66.2083}$ | 03/09/66 | 1000 1350 | Torejon | 1000 1350 | N/A | ${ }_{0}^{0.136+1 / 0.0082}$ | 0.163 | 0.000758 |
|  |  | $02111 / 86$ $02 / 1266$ | $02111 / 86$ $03 / 1 / 86$ | ${ }^{02111 / 66}$ | $66-2887$ $60-2031$ | 03/25/66 $03 / 16 / 86$ | 1350 960 | Torrejon Moron | 1350 960 | N/A | $0.105+/-0.102$ $0.114+/-0.081$ | 0.105 0.143 | 0.000714 |
|  |  | 02/13/66 | 03/19/8B | 03/02/36 | 66-2135 | 03/19/66 | 920 | Moron | 920 | N/A | $0.151+/-0.100$ | 0.197 | 0.000925 |
|  |  | 0227/66 | 03/19/86 | 03/09/68 | 86-2139 | 03/19/60 | 950 | Torrejon | 950 | N/A | $0.1302+/-0.1302$ | 0.164 |  |
|  |  | 011118/66 | 0204/36 | 01/26/96 | 66.355 | 02/4/166 | 560 | Torrelon | 200 | N/A | ND | ND |  |
|  |  | 0217108 | 03/10/38 | 02/27/66 | 86-2014 | 03/16/88 | 630 | Torrejon | 630 | N/A | $0.176+1-0.114$ | 0.335 |  |
|  |  | 01118/66 | 03103/66 | 02209/36 | 86.1930 | 03/03/68 | 845 | Albuquergue, NM | 845 | N/A | 0.183+1-0.093 | 0.260 |  |
|  |  | 01118/68 | 02008/66 | 01/27/86 | 66.513 | 02060666 | 640 | Moron | 200 | N/A | 4.53+\|-1.06 | 8.494 | 0.0259 |
|  |  | 01/18/68 | 0209/68 | 01/29/68 | 86.738 | 02<09168 | 970 | Hanaw, Germany | 200 | N/A | ND | ND |  |
|  |  | 01/11/68 | 03/20/86 | 02177166 | 86-2018 | 03/20/68 | 1320 | Tnker AFB | 1320 | N/A | 0.156+1-0.084 | 0.156 |  |
|  |  | 011/18/68 | 02006/66 | $01 / 27138$ | 68-525 | ${ }^{02208 / 86}$ | 480 | Torrejon | 200 | N/A | 1.44+1-0.77 | 3.757 | 0.0115 |
|  |  | 01123/86 | 03/20186 | 02120188 | 88.2305 | 03/20/66 | 880 | Pirmasens | 880 | N/A | $0.214+1 / 0.129$ | 0.292 | 0.00183 |
|  |  | 0221/68 | 03/18/88 | 03/05/66 | 66-2256 | 03/18/66 | 920 | Zaragoza | 920 | N/A | $0.225+1 / 0.134$ | 0.293 | 0.00109 |
|  |  | 011/18166 | 02/05/86 | 01/27786 | ${ }^{86-882}$ | ${ }^{022050566}$ | 950 | Moron | 200 | N/A | 1.22+1-0.73 | 1.541 | 0.00443 |
|  |  | 02211/65 | $03 / 18168$ | $03 / 05186$ | 86-2100 | 03/11/886 | 980 | Moron | 980 | N/A | $0.119+1 / 0.074$ | 0.146 | 0.000864 |
|  |  | 02131/66 | 03/19/66 | 03/02268 | 66-2229 | 03/19/66 | 500 | San Pablo | 500 | N/A | $0.226+1-0.134$ | 0.542 | 0.00184 |
|  |  | 02019/86 | 03/08/96 | 02/22/36 | 88-1901 | 03/0986 | 880 | Moron | 880 | N/A | $0.188+1 / 0.085$ | 0.254 |  |
|  |  | 01/18/66 | 01/21/38 | 01/19/68 | ${ }^{66-238}$ | 01/21/88 | 475 | Torrejon | 200 | N/A | NR | NR |  |
| (b) (6) |  | 011/18/66 | 0129/86 | 0123/96 | 66.241 66.1212 | $01 / 21 / 36$ $01 / 29 / 68$ | 350 880 | Torejon San Pablo | 200 | N/A | $\underset{\substack{\text { NR } \\ 0.427+/ \cdot 0.159}}{ }$ | NR 0.754 | 0.00108 |
|  | (b) (6) | 01/18/166 | 01/21/36 | 01/19/36 | 88.234 | 01/21/86 | 490 | Torrelon | 200 | N/A | ${ }^{0.42}$ | NR | 0.00100 |
|  |  | 02/24/68 | 03/19/66 | 03/07168 | 86-2168 | 03/19/66 | 1000 | Toul Rosleres | 1000 | N/A | 0.64 | 0.768 |  |
|  |  | 02061/60 | 02/26/66 | 0216/66 | 86.1424 | 02/26/66 | 860 | Moron | 894 | N/A | $0.205+/-0.124$ | 0.285 |  |
|  |  |  |  |  | 66-1424 | 02126/68 | 860 |  |  | 06/28/66 | NR | NR |  |
| (10) (6) | (b) (6) | 02113/66 | ${ }^{03143 / 36}$ | 02127166 | ${ }^{66.2678}$ | 03125/86 | 2000 | Torre]on | 2000 | N/A | $0.125+1 / 0.094$ | 0.125 |  |
|  |  | $011 / 17166$ | 0212156 | 01/30/68 | 68.777 | 0211266 | 292 | Torrejon | 200 | N/A | $0.49+1 / 0.45$ | 2.014 | 0.00791 |
|  |  | 011/1/68 | 02051/66 | ${ }^{01 / 27788}$ | ${ }^{66}$ 6-924 | ${ }^{02105 / 66}$ | 485 | Torrejon | 200 | N/A | 1.42+1-0.65 | 3.865 | 0.0105 |
|  |  | 01/22/66 | 02/20166 | 02/05/86 | 86-1040 | 02120/86 | 2200 | Furth, Ger. Us | 200 | 03/17/186 | 12.91+1-3.87 | 12.910 |  |
|  | (b) (6) |  |  |  |  |  |  | Army, 201 LSta Hosp. |  |  |  |  |  |
| (1b) (6) |  | 02/06/66 | 0226/36 | 0216/[6 | 86.1414 | 0226/68 | 640 | Hosp. Moron | 665 | N/A | 1.07+1/0.13 | 2.008 | 0.00454 |
|  |  | 0222166 | 03/99/68 | 03/08/86 | 88-2194 | 03/19/86 | 800 | Moron | 800 | N/A | ND | ND |  |
|  |  | 01/18/66 | 0128/86 | 01/23/68 | 66-3026 | 05106/36 | 1400 | Ramstein | 1400 | N/A | $0.131+1-0.057$ | 0.131 |  |
|  |  | 02108/68 | 03/19/86 | 0227/96 | 66-2183 | 03/19186 | 802 | Toul Rosieres | 800 | N/A | 0.829 | 1.240 |  |
|  |  | 01/18/66 | 0205/36 | 01/27/36 | 86-884 | 02005/66 | 880 | Torrejon | 200 | N/A | ND | ND |  |
|  |  | 0206166 | 0226/36 | 02148/36 | 68-1412 | 02126/86 | 650 | Moron | 676 | N/A | 0.135+1-0. 103 | 0.249 |  |
|  | (b) (6) | 03/44/86 | 03/28/56 | 03/20/66 | 66.1412 | 02126/86 | ${ }^{650}$ |  |  | 06/27768 | NR | NR |  |
| (b) (6) |  |  |  |  | ${ }_{66-2280}$ | 03/29986 | 710 | Torrejon | 710 | N/A | NO | ND |  |
|  |  |  |  |  | ${ }_{66}^{66.2865}$ | $04 / 08186$ | 1500 | Torrejon | 1500 | ${ }^{\text {N/A }}$ | $1.16{ }^{+1 / 0.28}$ | 1.160 |  |
| (6) | (b) (6) | 0209/88 | 03108/86 | 02/22768 | - $\begin{gathered}66-2865 \\ 68-1865\end{gathered}$ | -04/06/66 | 1500 800 | Moron | 800 | 08/30/66 | NR | NR ND |  |
| ( ${ }^{\text {( }}$ |  |  |  |  | 66-1865 | 03/08/88 | 800 |  |  | 07107166 | 0.273+1/0.220 | 0.410 |  |
| (16) (6) | (b) (6) | 01/18/66 | 02/09/86 | 01/29136 | 66-656 | 02/09/66 | 1950 | Blytheville AFB, AR | n/a | N/A | NR | NR |  |
| (b) (6) | (b) (G) | 01/188186 | 02205/36 | $01 / 27 / 188$ | ${ }^{86-922}$ | 0205/66 | 820 | Torrejon | 200 | N/A | NR | NR |  |
|  |  | 03/26166 | 04/11/88 | 04/03/68 | 66.3110 | 04/13/66 | ${ }^{625}$ | Moran | 625 | N/A | $0.330+1 / 0.087$ | 0.634 |  |
|  |  | 0211/66 | 03/12/66 | 02125/68 | 66-2641 | 03/28/66 | 600 |  | 600 | N/A | 0.287+/-0.168 | 0.574 |  |
| (b) (6) | (b) (6) | 02008166 | 02118/38 | 02112186 | ${ }^{66-1388}$ | 02118/66 | 700 |  | 728 | N/A | 1.05 | 1.800 | 0.004 |
|  |  | 01/18/66 | 0213/66 | 01/31/88 | 66-1441 | 03/02186 | 930 | Moron | 930 | 03/17/68 | ND | ND |  |
|  |  | 01/25/66 | 0209/56 | 02/01/88 | 86.811 | 02109/66 | 810 | Hanaw, Germany | 200 | N/A | $1.164+1.74$ | 2.430 | 0.00873 |
|  |  | 01/17166 | 0226/36 | 02108/36 | B6. 1418 | 02126/66 | 860 | Torrejon | 915 | N/A | 0.117+/-0.157 | 0.160 |  |
|  |  | 01/18166 | 01/17/66 | 01/17/86 | 66-1100 | 01117/86 | 1180 | Torrejon | 1180 | 03/17768 | ND | ND |  |
|  |  | 03/26/66 | 03/29/68 | 03/27/86 | 86.3124 | 04/13/86 | 980 | Moron | 490 | N/A | 0.212+/.0.161 | 0.260 |  |
| (b) (6) | (13) (6) |  |  |  | 66-3124-S | 04/13/66 | 980 | Moron | 490 | N/A | NR | NR |  |
|  |  | 01/24/68 | 0130/36 | 01127156 | ${ }^{68 \cdot 2688}$ | 03/281866 | 1800 | Torrelon | 1800 | N/A | $0.112+$-0. 101 | 0.112 |  |
|  |  | $021 / 18168$ $01 / 18188$ | $03 / 108168$ $0204 / 66$ | - $01 / 2726868186$ | ${ }^{86-2472}$ | (03108/86 | 900 950 | Torajon Moron | 900 200 | N/A | $\stackrel{0.194}{\text { ND }}$ | 0.259 |  |
|  |  |  |  |  |  |  |  |  |  |  |  |  |  |




Individuals with Urino Samples Classified as Remaining Cases





[^0]:    Release of this document is restricted under tho provisions of the Privacy Act, 5 U.S.C. 552(a). C.2-272

[^1]:    Release of this document is restricted under the provisions of the Privacy Act, 5 U.S.C. 552(a). C.2-278

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[^3]:    Release of this document is restricted under the provisions of the Privacy Act, 5 U.S.C. 552(a). C.2-281

[^4]:    Release of this document is restricted under the provisions of the Privacy Act, 5 U.S.C. 552(a). C.2-335

[^5]:    Release of this document is restricted under the provisinns of the Privacy Act, 5 U.S.C. 552(a). C.2-358

[^6]:    Release of this document is restricted under the provisinns of the Privacy Act, 5 U.S.C. 552(a). C.2-364

[^7]:    Release of this document is restricted under the provisings of the Privacy Act, 5 U.S.C. 552(a). C.2-370

[^8]:    Release of this document is restricted under the provisions of the Privacy Act, 5 U.S.C. 552(a). C.2-375

[^9]:    NR - Not Reported; ND - No Detectable Activity; N/a - not avalatle

[^10]:    Palomares Nuclear Weapons Accident

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[^12]:    Palomares Nuclear Weapons Accident DRAFT
    

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